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NUCLEAR REGULATORY COMMISSION

10 CFR Part 50

[Docket Nos. PRM-50-108; NRC-2014-0171]

Fuel-Cladding Issues in Postulated Spent Fuel Pool Accidents

AGENCY: Nuclear Regulatory Commission.

ACTION: Petition for rulemaking; denial.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is denying a petition for rulemaking (PRM or the petition), PRM-50-108, submitted by Mr. Mark Edward Leyse (the petitioner). The petitioner requested that the NRC require power reactor licensees to perform evaluations to determine the potential consequences of various postulated spent fuel pool (SFP) accident scenarios. The evaluations would be required to be submitted to the NRC for informational purposes. The NRC is denying the petition because the NRC does not believe the information is needed for effective NRC regulatory decisionmaking with respect to SFPs or for public safety, environmental protection, or common defense and security.

DATES: The docket for the petition, PRM-50-108, is closed on May 13, 2016.

ADDRESSES: Please refer to Docket ID NRC-2014-0171 when contacting the NRC about the availability of information for this petition. You may obtain publicly-available information related to this petition by any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2014-0171. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: Carol.Gallagher@nrc.gov. For

technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *The NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Document collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-Based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in the **SUPPLEMENTARY INFORMATION** section. For the convenience of the reader, instructions about obtaining materials referenced in this document are provided in Section IV, "Availability of Documents," of this document.

- *The NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT: Daniel Doyle, Office of Nuclear Reactor Regulation; U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; telephone: 301-415-3748; email: Daniel.Doyle@nrc.gov.

SUPPLEMENTARY INFORMATION:

Table of Contents

- I. The Petition
- II. Reasons for Denial
- III. Conclusion
- IV. Availability of Documents

I. The Petition

Section 2.802 of title 10 of the *Code of Federal Regulations* (10 CFR), "Petition for rulemaking—requirements for filing," provides an opportunity for any interested person to petition the Commission to issue, amend, or rescind any regulation. The NRC received a petition dated June 19, 2014, from Mr. Mark Edward Leyse and assigned it Docket No. PRM-50-108 (ADAMS Accession No. ML14195A388). The NRC published a notice of docketing in the **Federal Register** (FR) on October 7, 2014 (79 FR 60383). The NRC did not request public comment on the petition

because sufficient information was available for the NRC staff to form a technical opinion regarding the merits of the petition.

The petitioner requested that the NRC develop new regulations requiring that: (1) SFP accident evaluation models use data from multi-rod bundle (assembly) severe accident experiments for calculating the rates of energy release, hydrogen generation, and fuel cladding oxidation from the zirconium-steam reaction; (2) SFP accident evaluation models use data from multi-rod bundle (assembly) severe accident experiments conducted with pre-oxidized fuel cladding for calculating the rates of energy release (from both fuel cladding oxidation and fuel cladding nitriding), fuel cladding oxidation, and fuel cladding nitriding from the zirconium-air reaction; (3) SFP accident evaluation models be required to conservatively model nitrogen-induced breakaway oxidation behavior; and (4) licensees be required to use conservative SFP accident evaluation models to perform annual SFP safety evaluations of: postulated complete loss-of-coolant accident (LOCA) scenarios, postulated partial LOCA scenarios, and postulated boil-off accident scenarios.

The petitioner referenced recent NRC post-Fukushima MELCOR simulations of boiling-water reactor Mark I SFP accident/fire scenarios. The petitioner stated that the conclusions from the NRC's MELCOR simulations are non-conservative and misleading because their conclusions underestimate the probabilities of large radiological releases from SFP accidents.

The petitioner asserted that in actual SFP fires, there would be quicker fuel-cladding temperature escalations, releasing more heat, and quicker axial and radial propagation of zirconium (Zr) fires than MELCOR simulations predict. The petitioner stated that the NRC's philosophy of defense-in-depth requires the application of conservative models, and, therefore, it is necessary to improve the performance of MELCOR and any other computer safety models that are intended to accurately simulate SFP accident/fire scenarios.

The petitioner stated that the new regulations would help improve public and plant-worker safety. The petitioner asserted that the first three requested regulations, regarding zirconium fuel cladding oxidation and nitriding, as

well as nitrogen-induced breakaway oxidation behavior, are intended to improve the performance of computer safety models that simulate postulated SFP accident/fire scenarios. The petitioner stated that the fourth requested regulation would require that licensees use conservative SFP accident evaluation models to perform annual SFP safety evaluations of postulated complete LOCA scenarios, postulated partial LOCA scenarios, and postulated boil-off accident scenarios. The petitioner stated that the purpose of these evaluations would be to keep the NRC informed of the potential consequences of postulated SFP accident/fire scenarios as fuel assemblies were added, removed, or reconfigured in licensees' SFPs. The petitioner stated that the requested regulations are needed because the probability of the type of events that could lead to SFP accidents is relatively high.

The NRC staff reviewed the petition and, based on its understanding of the overall argument in the petition, identified and evaluated the following three issues:

- Issue 1: The requested regulations pertaining to SFP accident evaluation models are needed because the probability of the type of events that could lead to SFP accidents is relatively high.
- Issue 2: Annual licensee SFP safety evaluations and submission of results to the NRC is necessary so that the NRC is aware of potential consequences of postulated SFP accident/fire scenarios as fuel assemblies are added, removed, or reconfigured in licensees' SFPs.
- Issue 3: MELCOR is not currently sufficient to provide a conservative evaluation of postulated SFP accident/fire scenarios for use in the PRM-proposed annual SFP evaluations.

Detailed NRC responses to the three issues are provided in Section II, "Reasons for Denial," of this document.

II. Reasons for Denial

The NRC is denying the petition because the petitioner failed to present any significant information or arguments that would warrant the requested regulations. The first three requested regulations would establish requirements for how the detailed annual evaluations that would be required by the fourth requested regulation would be performed. It is not necessary to require detailed annual evaluations of the progression of SFP severe accidents because the risk of an SFP severe accident is low. The NRC defines risk as the product of the probability and the consequences of an accident. The requested annual

evaluations are not needed for regulatory decisionmaking, and the evaluations would not prevent or mitigate an SFP accident. The petitioner described multiple ways that an extended loss of offsite electrical power could occur and how this could lead to an SFP fire. In order for an SFP fire to occur, all SFP systems, backup systems, and operator actions that are intended to prevent the spent fuel in the pool from being uncovered would have to fail. The NRC does not agree that more detailed accident evaluation models need to be developed for this purpose, as requested by the petitioner, because the requested annual evaluations are not needed for regulatory decisionmaking. The NRC recognizes that the consequences of an SFP fire could be large and that is why there are numerous requirements in place to prevent a situation where the spent fuel is uncovered.

This section provides detailed NRC responses to the three issues identified in the petition.

Issue 1: The Requested Regulations Pertaining to SFP Accident Evaluation Models Are Needed Because the Probability of the Type of Events That Could Lead to SFP Accidents Is Relatively High

The petitioner stated that the requested regulations pertaining to SFP accident evaluation models are needed because the probability of the type of events that could lead to SFP accidents is relatively high. The petitioner stated that an SFP accident could happen as a result of a leak (rapid drain down) or boil-off scenario. Furthermore, the petitioner notes that in the event of a long-term station blackout, emergency diesel generators could run out of fuel and SFP cooling would be lost, resulting in a boil-off of SFP water inventory and a subsequent release of radioactive materials from the spent fuel. The petitioner also provided several examples of events that could lead to a long-term station blackout and, ultimately, an SFP accident, such as a strong geomagnetic disturbance, a nuclear device detonated in the earth's atmosphere, a pandemic, or a cyber or physical attack.

NRC Response

Spent nuclear fuel offloaded from a reactor is initially stored in an SFP. The SFPs at all nuclear plants in the United States are robust structures constructed with thick, reinforced, concrete walls and welded stainless-steel liners. They are designed to safely contain the spent fuel discharged from a nuclear reactor under a variety of normal, off-normal, and hypothetical accident conditions

(e.g., loss of electrical power, loss of cooling, fuel or cask drop incidents, floods, earthquakes, or extreme weather events). Racks fitted in the SFPs store the fuel assemblies in a controlled configuration so that the fuel is maintained in a sub-critical and coolable geometry. Redundant monitoring, cooling, and water makeup systems are provided. The spent fuel assemblies are typically covered by at least 25-feet of water, which provides passive cooling as well as radiation shielding. Penetrations to pools are limited to prevent inadvertent drainage, and the penetrations are generally located well above spent fuel storage elevations to prevent uncovering of fuel from drainage.

Studies conducted over the last four decades have consistently shown the risk of an accident causing a zirconium fire in an SFP to be low. The risk of an SFP accident was examined in the 1980s as Generic Issue 82, "Beyond Design Basis Accidents in Spent Fuel Pools," in light of increased use of high-density storage racks and laboratory studies that indicated the possibility of zirconium fire propagation between assemblies in an air-cooled environment (Section 3 of NUREG-0933, "Resolution of Generic Safety Issues," <http://nureg.nrc.gov/sr0933/>). The risk assessment and cost-benefit analyses developed through this effort, Section 6.2 of NUREG-1353, "Regulatory Analysis for the Resolution of Generic Issue 82, Beyond Design Basis Accidents in Spent Fuel Pools" (ADAMS Accession No. ML082330232), concluded that the risk of a severe accident in the SFP was low and appeared to meet the objectives of the Commission's Safety Goal Policy Statement public health objectives (51 FR 30028; August 21, 1986) and that no new regulatory requirements were warranted.

The risk of an SFP accident was reassessed in the late 1990s to support a risk-informed rulemaking for permanently shutdown, or decommissioned, nuclear power plants in the United States. The study, NUREG-1738, "Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants" (ADAMS Accession No. ML010430066), conservatively assumed that if the water level in the SFP dropped below the top of the spent fuel, an SFP zirconium fire involving all of the spent fuel would occur, and thereby bounded those conditions associated with air cooling of the fuel (including partial-drain down scenarios) and fire propagation. Even with this conservative assumption, the study

found the risk of an SFP fire to be low and well within the Commission's Safety Goals.

Additional mechanisms to mitigate the potential loss of SFP water inventory were implemented following the terrorist attacks of September 11, 2001, which have enhanced spent fuel coolability and the potential to recover SFP water level and cooling prior to a potential SFP zirconium fire (73 FR 76204; August 8, 2008). Based on the implementation of these additional strategies, the probability and, accordingly, the risk of an SFP zirconium fire initiation has decreased and is expected to be less than previously analyzed in NUREG-1738 and previous studies.

Following the 2011 accident at Fukushima Dai-ichi, the NRC took extensive actions to ensure that portable equipment is available to mitigate a loss of cooling water in the SFP. On March 12, 2012, the NRC issued Order EA-12-049, "Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events" (ADAMS Accession No. ML12054A735). This order required licensees to develop, implement, and maintain guidance and strategies to maintain or restore core cooling, containment, and SFP cooling capabilities following a beyond-design-basis external event. The NRC endorsed the Nuclear Energy Institute (NEI) guidance to meet the requirements of this order.¹ That guidance establishes additional mechanisms for mitigating a loss of SFP cooling water beyond the requirements in 10 CFR 50.54(hh)(2), such as installing a remote connection for SFP makeup water that can be accessed away from the SFP refueling floor.

Also, in 2014, the NRC documented a regulatory analysis in COMSECY-13-0030, "Staff Evaluation and Recommendation for Japan Lessons Learned Tier 3 Issue on Expedited Transfer of Spent Fuel" (ADAMS Accession No. ML13329A918), which considered a broad history of the NRC's oversight of spent fuel storage, SFP operating experience (domestic and international), as well as information compiled in NUREG-2161, "Consequence Study of a Beyond-Design-Basis Earthquake Affecting the

Spent Fuel Pool for a U.S. Mark I Boiling Water Reactor" (ADAMS Accession No. ML14255A365). In COMSECY-13-0030, the NRC staff concluded that SFPs are robust structures with large safety margins and recommended to the Commission that assessments of possible regulatory actions to require the expedited transfer of spent fuel from SFPs to dry cask storage were not warranted. The Commission subsequently approved the staff's recommendation in the Staff Requirements Memorandum to COMSECY-13-0030 (ADAMS Accession No. ML14143A360).

As supported by numerous evaluations referenced in this document, the NRC has determined that the risk of an SFP severe accident is low. While the risk of a severe accident in an SFP is not negligible, the NRC believes that the risk is low because of the conservative design of SFPs; operational criteria to control spent fuel movement, monitor pertinent parameters, and maintain cooling capability; mitigation measures in place if there is loss of cooling capability or water; and emergency preparedness measures to protect the public. The information proposed to be provided to the NRC is not needed for the effectiveness of NRC's approach for ensuring SFP safety. The NRC notes that the issue of long-term cooling of SFPs is the subject of PRM-50-96, which was accepted for consideration in the rulemaking process (77 FR 74788; December 18, 2012) and is being addressed by the NRC's rulemaking regarding mitigation of beyond design-basis events (RIN 3150-AJ49; NRC-2014-0240).

Issue 2: Annual Licensee SFP Safety Evaluations and Submission of Results to the NRC Is Necessary So That the NRC Is Aware of Potential Consequences of Postulated SFP Accident/Fire Scenarios as Fuel Assemblies Are Added, Removed, or Reconfigured in Licensees' SFPs

The petitioner stated that the purpose of the proposed requirement is to keep the NRC informed of the potential consequences of postulated SFP accident/fire scenarios as fuel assemblies are added, removed, or reconfigured in licensees' SFPs.

NRC Response

The NRC does not agree that this is necessary because the NRC already evaluates SFP systems and structures during initial licensing and license amendment reviews. In addition, baseline NRC inspections provide ongoing oversight to ensure adequate

protection. There are not sufficient benefits that would justify the new requirement proposed in the petition for SFP accident evaluations. The proposed new requirement for licensees to perform SFP evaluations would not prevent or mitigate an SFP accident or provide information that is necessary for regulatory decisionmaking. The annual licensee SFP safety evaluations and their results proposed to be provided to the NRC are not needed for the effectiveness of the NRC's approach to ensuring SFP safety.

The NRC issues licenses after reviewing and approving the design and licensing bases contained in the plant's safety analysis report. Licensees are required to operate the plant, including performing operations and surveillances related to spent fuel, in accordance with technical specifications and established practices and procedures for that plant. Any licensee changes to design, operational or surveillance practices, or approved spent fuel inventory limits or configuration changes must be evaluated using the criteria in 10 CFR 50.59, documented and retained for the duration of the operating license, and, if warranted, submitted to the NRC for prior approval.

The general design criteria (GDC) in appendix A to 10 CFR part 50 establish general expectations that licensees must meet through compliance with their plant-specific licensing basis. Several GDC apply to SFPs:

- Protecting against natural phenomena and equipment failures (GDC 2 and GDC 4);
- Preventing a substantial loss-of-coolant inventory under accident conditions (e.g., equipment failure or loss of decay and residual heat removal) (GDC 61);
- Preventing criticality of the spent fuel (GDC 62); and
- Adequately monitoring the SFP conditions for loss of decay heat removal and radiation (GDC 63).

Additionally, emergency procedures and mitigating strategies are in place to address unexpected challenges to spent fuel safety. Multiple requirements in 10 CFR part 50, as well as recent NRC orders following the Fukushima Dai-ichi accident, require redundant equipment and strategies to address loss of cooling to SFPs and protective actions for plant personnel and the public to limit exposure to radioactive materials.

The NRC provides oversight of the licensee's overall plant operations and the SFP in several ways. The NRC inspectors ensure that spent fuel is stored safely by regularly inspecting reactor and equipment vendors; inspecting the design, construction, and

¹ See NEI 12-06, "Diverse and Flexible Coping Strategies (FLEX) Implementation Guide," dated August 2012 (ADAMS Accession No. ML12242A378), and JLD-ISG-2012-01, "Compliance with Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events," dated August 2012 (ADAMS Accession No. ML12229A174).

use of equipment; and observing “dry runs” of procedures. At least two NRC resident inspectors are assigned to each site to provide monitoring and inspection of routine and special activities. They are aware of, and routinely observe, SFP activities involving fuel manipulation. The NRC inspectors use inspection procedures to guide periodic inspection activities, and the results are published in publicly-available inspection reports. Special inspections may be conducted, as necessary, to evaluate root causes and licensee corrective actions if site-specific events occur. Special inspections may also evaluate generic actions taken by some or all licensees as a result of an NRC order or a change in regulations.

In accordance with 10 CFR part 21, the NRC is informed of defects and noncompliances associated with basic components, which include SFPs and associated drain pipes and safety-related systems, structures, and components for makeup water. This information allows the NRC to take additional regulatory action as necessary with respect to defects and noncompliances. The NRC is also informed of events and conditions at nuclear power plants, as set forth in §§ 50.72 and 50.73. Depending upon the nature of the event or condition, a nuclear power plant licensee must inform the NRC within a specified period of time of the licensee’s corrective action taken or planned to be taken. These reports also facilitate effective and timely NRC regulatory oversight. Finally, information identified by a nuclear power plant applicant or licensee as having a significant implication for public health and safety or common defense and security must be reported to the NRC within 2 days of the applicant’s or licensee’s identification of the information.

The annual evaluations requested in the petition would not provide information that is necessary for regulatory decisionmaking. The evaluations requested in the petition would postulate scenarios in which the normal cooling systems, the backup cooling methods, and the mitigation strategies have all failed to cool the stored fuel and would require the calculation of the time it would take for the stored fuel to ignite and how much of it would ignite. Due to the robustness of this equipment, the NRC views this sequence of events as extremely unlikely to occur. Since the current regulations require that the pool be designed to prevent the loss-of-coolant and subsequent uncovering of the fuel, the information that would be obtained

from the proposed requirement in the petition would not impact the current design basis. Moreover, as discussed previously, the NRC’s current regulatory infrastructure relevant to SFPs at nuclear power plants in the United States already contains information collection and reporting requirements that support effective NRC regulatory oversight of SFPs.

The NRC does not agree that it is necessary to impose a new requirement for licensees to perform annual evaluations of their SFPs because existing requirements and oversight are sufficient to ensure adequate protection of public health and safety.

Issue 3: MELCOR Is Not Currently Sufficient To Provide a Conservative Evaluation of Postulated SFP Accident/Fire Scenarios

The petitioner requested that the NRC establish requirements for SFP accident evaluation computer models to be used in the annual SFP evaluations requested in Issue 2. The petitioner stated that there are serious flaws with MELCOR, which has been used by the NRC to model severe accident progression in SFPs, and, therefore, MELCOR is not sufficient.

NRC Response

The NRC does not agree that it is necessary to establish requirements for SFP accident evaluation computer models because the annual SFP evaluations requested in Issue 2 are not necessary for regulatory decisionmaking. Therefore, it is not necessary for the NRC to establish requirements for how such an evaluation should be conducted. Furthermore, the NRC disagrees with the petitioner’s statements that MELCOR is flawed.

There are inherent uncertainties in the progression of severe accidents. There are many interrelated phenomena that need to be properly understood; otherwise, conservatism in one area may lead to overall non-conservative results. Conservatism can be meaningfully introduced into the relevant analysis after the best estimate analysis is done and uncertainties are properly taken into account.

The important question for a severe accident analysis is whether the uncertainties are appropriately considered in the analysis results. For example, Section 9 of the SFP study (NUREG–2161) is devoted to discussing the major uncertainties that can affect the radiological releases (e.g., hydrogen combustion, core concrete interaction, multi-unit or concurrent accident, or fuel loading). In addition, the regulatory

analysis in COMSECY–13–0030 only relied on SFP study insights for the boiling-water reactors with Mark I and II containments, and, even then, the results were conservatively biased towards higher radiological releases. For other designs, the release fractions were based on previous studies (i.e., NUREG–1738) that used bounding or conservative estimates.

The MELCOR computer code is the NRC’s best estimate tool for severe accident analysis. It has been validated against experimental data, and it represents the current state of the art in severe accident analysis. In NUREG–2161, the NRC stated that “MELCOR has been developed through the NRC and international research performed since the accident at Three Mile Island in 1979. MELCOR is a fully integrated, engineering-level computer code and includes a broad spectrum of severe accident phenomena with capabilities to model core heatup and degradation, fission product release and transport within the primary system and containment, core relocation to the vessel lower head, and ex-vessel core concrete interaction.” Furthermore, MELCOR has been benchmarked against many experiments, including separate and integral effects tests for a wide range of phenomena. Therefore, the NRC has determined that MELCOR is acceptable for its intended use.

Additional information about the capabilities of the MELCOR code to model SFP accidents can be found in the NRC response to stakeholder comments in Appendix E to NUREG–2161. The NRC also addressed questions regarding MELCOR in Appendix D to NUREG–2157, Volume 2, “Generic Environmental Impact Statement for Continued Storage of Spent Nuclear Fuel” (ADAMS Accession No. ML14196A107).

III. Conclusion

For the reasons described in Section II, “Reasons for Denial,” of this document, the NRC is denying the petition under 10 CFR 2.803. The petitioner failed to present any information or arguments that would warrant the requested amendments. The NRC does not believe that the information that would be reported to the NRC as requested by the petitioner is necessary for effective NRC regulatory decisionmaking with respect to SFPs. The NRC continues to conclude that the current design and licensing requirements for SFPs provide adequate protection of public health and safety.

IV. Availability of Documents

The documents identified in the following table are available to

interested persons as indicated. For more information on accessing ADAMS,

see the **ADDRESSES** section of this document.

Date	Document	ADAMS accession number/ Federal Register citation
August 21, 1986	Safety Goals for the Operations of Nuclear Power Plants; Policy Statement; Republication.	51 FR 30028.
April 1989	NUREG-1353, "Regulatory Analysis for the Resolution of Generic Issue 82, Beyond Design Basis Accidents in Spent Fuel Pools".	ML082330232.
February 2001	NUREG-1738, "Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants".	ML010430066.
March 12, 2012	EA-12-049, "Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events".	ML12054A735.
August 2012	NEI 12-06, "Diverse and Flexible Coping Strategies (FLEX) Implementation Guide".	ML12242A378.
August 2012	JLD-ISG-2012-01, "Compliance with Order EA-12-049, Order Modifying Licenses with Regard to Requirements for Mitigation Strategies for Beyond-Design-Basis External Events".	ML12229A174.
December 18, 2012	Long-Term Cooling and Unattended Water Makeup of Spent Fuel Pools ...	77 FR 74788.
November 12, 2013	COMSECY-13-0030, "Staff Evaluation and Recommendation for Japan Lessons Learned Tier 3 Issue on Expedited Transfer of Spent Fuel".	ML13329A918.
May 23, 2014	SRM-COMSECY-13-0030, "Staff Requirements—COMSECY-13-0030—Staff Evaluation and Recommendation for Japan Lessons-Learned Tier 3 Issue on Expedited Transfer of Spent Fuel".	ML14143A360.
June 19, 2014	Incoming Petition (PRM-50-108) from Mr. Mark Edward Leyse	ML14195A388.
September 2014	NUREG-2157, "Generic Environmental Impact Statement for Continued Storage of Spent Nuclear Fuel," Volume 2.	ML14196A107.
September 2014	NUREG-2161, "Consequence Study of a Beyond-Design-Basis Earthquake Affecting the Spent Fuel Pool for a U.S. Mark I Boiling-Water Reactor".	ML14255A365.
October 7, 2014	Notice of Docketing for PRM-50-108	79 FR 60383.

Dated at Rockville, Maryland, this 5th day of May, 2016.

For the Nuclear Regulatory Commission.

Annette L. Vietti-Cook,

Secretary of the Commission.

[FR Doc. 2016-11212 Filed 5-12-16; 8:45 am]

BILLING CODE 7590-01-P

PENSION BENEFIT GUARANTY CORPORATION

29 CFR Parts 4010, 4041, 4071, and 4302

RIN 1212-AB33

Adjustment of Civil Penalties

AGENCY: Pension Benefit Guaranty Corporation.

ACTION: Interim final rule.

SUMMARY: The Pension Benefit Guaranty Corporation is amending its regulations to adjust the penalties provided for in sections 4071 and 4302 of the Employee Retirement Income Security Act of 1974. This action is being taken in accordance with the Federal Civil Penalties Inflation Adjustment Act Improvements Act of 2015 and Office of Management and Budget memorandum M-16-06. The regulations being amended are those on Penalties for Failure to Provide Certain Notices or Other Material Information (29 CFR part 4071) and

Penalties for Failure to Provide Certain Multiemployer Plan Notices (29 CFR part 4302). Conforming amendments are also being made to the regulations on Annual Financial and Actuarial Information Reporting (29 CFR part 4010) and Termination of Single-Employer Plans (29 CFR part 4041).

DATES: The amendments are effective August 1, 2016. Also see Applicability, below.

FOR FURTHER INFORMATION CONTACT:

Deborah C. Murphy, Deputy Assistant General Counsel for Regulatory Affairs (murphy.deborah@pbgc.gov), Office of the General Counsel, Pension Benefit Guaranty Corporation, 1200 K Street NW., Washington, DC 20005-4026; 202-326-4400 extension 3451. (TTY and TDD users may call the Federal relay service toll-free at 800-877-8339 and ask to be connected to 202-326-4400 extension 3451.)

SUPPLEMENTARY INFORMATION:

Executive Summary

Purpose of the Regulatory Action

This rule is needed to carry out the requirements of the Federal Civil Penalties Inflation Adjustment Act Improvements Act of 2015. The rule adjusts the maximum civil penalties that PBGC may assess for failure to provide certain notices or other material information.

PBGC's legal authority for this action comes from the Federal Civil Penalties Inflation Adjustment Act of 1990 as amended by the Federal Civil Penalties Inflation Adjustment Act Improvements Act of 2015 and from sections 4002(b)(3), 4071, and 4302 of the Employee Retirement Income Security Act of 1974.

Major Provisions of the Regulatory Action

This rule adjusts the maximum civil penalties that PBGC may assess under sections 4071 and 4302 of ERISA. The new maximum amounts are \$2,063 for section 4071 penalties and \$275 for section 4302 penalties.

Background

The Pension Benefit Guaranty Corporation (PBGC) administers title IV of the Employee Retirement Income Security Act of 1974 (ERISA). Title IV has two provisions that authorize PBGC to assess civil monetary penalties.¹ Section 4302, added to ERISA by the Multiemployer Pension Plan

¹ Under the Federal Civil Penalties Inflation Adjustment Act of 1990, a penalty is a civil monetary penalty if (among other things) it is for a specific monetary amount or has a maximum amount specified by Federal law. Title IV also provides (in section 4007) for penalties for late payment of premiums, but those penalties are neither in a specified amount nor subject to a specified maximum amount.