

The increase in radiation levels due to the proposed power uprate under post-accident conditions has no significant effect on the plant, or on the habitability of the Technical Support Center or Emergency Operations Facility.

The licensee re-evaluated the effect of the power uprate on Design Basis Accident (DBA) radiological consequences and reported these results (Section 9.2 of Reference 14). The original licensing DBA source terms for Brunswick were considered. The licensee also re-evaluated the control room habitability under DBA conditions. The licensee stated that the radiological consequence analyses were performed using standard models developed by GE that have been utilized in other power uprate projects. The dose analyses were based on plant-specific parameters from the BSEP Updated Final Safety Analysis Report and were calculated at both the current power and at 102% of the proposed uprate power. The licensee's analyses indicate that the calculated offsite radiological consequences doses for all DBAs are within the dose acceptance criteria stated in the NRC's Standard Review Plan (SRP) and 10 CFR Part 100 and also comply with the dose acceptance criteria for control room operators given in General Design Criterion (GDC) 19 of Appendix A to 10 CFR Part 50. The staff performed confirmatory evaluations of radiological consequences of DBAs for the proposed power uprate. The staff found that the offsite radiological consequences and control room operator doses for all DBAs at the uprated power level of 2558 MWt will continue to meet the acceptance criteria of the SRP, 10 CFR Part 100, and GDC 19.

The NRC staff finds the licensee's assessment of the radiological effects of the proposed action acceptable and concludes that the proposed uprate will not significantly increase radiological impacts on the environment.

Alternatives to the Proposed Action

Since the Commission has concluded there is no significant (within existing limits) environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the proposed action would result in no change in current environmental impacts of plant operation, but would restrict operation of BSEP to the currently licensed power level. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the BSEP.

Agencies and Persons Consulted

In accordance with its stated policy, on October 17, 1996, the staff consulted with the North Carolina State official, Mr. J. James, of the North Carolina Department of Environment, Commerce and Natural Resources, Division of Radiation Protection, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action. For further details with respect to the proposed action, see the licensee's application dated April 2, 1996, as supplemented by an earlier submittal dated November 20, 1995, and by subsequent submittals dated July 1, 1996, July 30, 1996, August 7, 1996, September 13, 1996, September 20, 1996, October 1, 1996, October 22, 1996 (BSEP 96-0392), and October 22, 1996 (BSEP 96-0403), which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the University of North Carolina at Wilmington, William Madison Randall Library, 601 College Road, Wilmington, North Carolina 28403-3297.

References

1. GE Nuclear Energy, "Generic Guidelines For General Electric Boiling Water Reactor Power Uprate," Licensing Topical Report NEDO-31897, Class I (non-proprietary), February 1992; and NEDC-31897P-A, Class III (Proprietary), May 1992.
2. W. T. Russell, U.S. Nuclear Regulatory Commission, letter to P.W. Marriott, General Electric Company, "Staff Position Concerning General Electric Boiling Water Reactor Power Uprate Program," September 30, 1991.
3. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, "105% Thermal Power Uprate," April 2, 1996.
4. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, "Power Uprate," November 20, 1995.

5. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, July 1, 1996.
6. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, July 30, 1996.
7. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, August 7, 1996.
8. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, September 13, 1996.
9. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, September 20, 1996.
10. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, October 1, 1996.
11. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, October 22, 1996 (BSEP 96-0392).
12. W. R. Campbell, Carolina Power & Light Company, letter to U.S. Nuclear Regulatory Commission, October 22, 1996 (BSEP 96-0403).
13. "Final Environmental Statement," Brunswick Steam Electric Plant dated January 1974.
14. GE Nuclear Energy, "Power Uprate Safety Analysis Report for Brunswick Steam Electric Plant, Units 1 & 2," Licensing Topical Report NEDC-32466P, Class III (Proprietary), September 1995 (Enclosure 1 to Ref. 4); NEDO-32466, Class I (Non-proprietary) September 1995 (Enclosure 2 to Ref. 4).

Dated at Rockville, Maryland, this 23rd day of October, 1996.

For the Nuclear Regulatory Commission.

Bartholomew C. Buckley,

Acting Director, Project Directorate II-1, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

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Draft Regulatory Guide; Issuance, Availability

The Nuclear Regulatory Commission has issued for public comment a draft of a guide planned for its Regulatory Guide Series. This series has been developed to describe and make available to the public such information as methods acceptable to the NRC staff for implementing specific parts of the Commission's regulations, techniques used by the staff in evaluating specific problems or postulated accidents, and data needed by the staff in its review of applications for permits and licenses.

The draft guide is a proposed Revision 3 to Regulatory Guide 1.105, and it is temporarily identified as DG-1045, "Setpoints for Safety-Related Instrumentation." The guide will be in Division 1, "Power Reactors." This regulatory guide is being revised to

provide current guidance on methods acceptable to the NRC staff for complying with the NRC's regulations for ensuring that setpoints for safety-related instrumentation are initially within and remain within the licensee's technical specification limits. The guide will endorse Part 1 of an Instrument Society of America standard, ISA-S67.04-1994, "Setpoints for Nuclear Safety-Related Instrumentation."

The draft guide has not received complete staff review and does not represent an official NRC staff position.

Public comments are being solicited on the guide. Comments should be accompanied by supporting data. Written comments may be submitted to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555. Copies of comments received may be examined at the NRC Public Document Room, 2120 L Street NW., Washington, DC. Comments will be most helpful if received by December 31, 1996.

Although a time limit is given for comments on this draft guide, comments and suggestions in connection with items for inclusion in guides currently being developed or improvements in all published guides are encouraged at any time.

Comments may be submitted electronically, in either ASCII text or Wordperfect format (version 5.1 or later), by calling the NRC Electronic Bulletin Board on FedWorld. The bulletin board may be accessed using a personal computer, a modem, and one of the commonly available communications software packages, or directly via Internet.

If using a personal computer and modem, the NRC subsystem on FedWorld can be accessed directly by dialing the toll free number: 1-800-303-9672. Communication software parameters should be set as follows: parity to none, data bits to 8, and stop bits to 1 (N,8,1). Using ANSI or VT-100 terminal emulation, the NRC NUREGs and RegGuides for Comment subsystem can then be accessed by selecting the "Rules Menu" option from the "NRC Main Menu." For further information about options available for NRC at FedWorld, consult the "Help/Information Center" from the "NRC Main Menu." Users will find the "FedWorld Online User's Guides" particularly helpful. Many NRC subsystems and data bases also have a "Help/Information Center" option that is tailored to the particular subsystem.

The NRC subsystem on FedWorld can also be accessed by a direct dial phone

number for the main FedWorld BBS, 703-321-3339, or by using Telnet via Internet, fedworld.gov. If using 703-321-3339 to contact FedWorld, the NRC subsystem will be accessed from the main FedWorld menu by selecting the "Regulatory, Government Administration and State Systems," then selecting "Regulatory Information Mall." At that point, a menu will be displayed that has an option "U.S. Nuclear Regulatory Commission" that will take you to the NRC Online main menu. The NRC Online area also can be accessed directly by typing "/go nrc" at a FedWorld command line. If you access NRC from FedWorld's main menu, you may return to FedWorld by selecting the "Return to FedWorld" option from the NRC Online Main Menu. However, if you access NRC at FedWorld by using NRC's toll-free number, you will have full access to all NRC systems but you will not have access to the main FedWorld system.

If you contact FedWorld using Telnet, you will see the NRC area and menus, including the Rules menu. Although you will be able to download documents and leave messages, you will not be able to write comments or upload files (comments). If you contact FedWorld using FTP, all files can be accessed and downloaded but uploads are not allowed; all you will see is a list of files without descriptions (normal Gopher look). An index file listing all files within a subdirectory, with descriptions, is included. There is a 15-minute time limit for FTP access.

Although FedWorld can be accessed through the World Wide Web, like FTP that mode only provides access for downloading files and does not display the NRC Rules menu.

For more information on NRC bulletin boards call Mr. Arthur Davis, Systems Integration and Development Branch, U.S. Nuclear Regulatory Commission, Washington, DC 20555, telephone (301)415-5780; e-mail AXD3@nrc.gov. For more information on this draft regulatory guide, contact S.K. Aggarwal at the NRC, telephone (301) 415-6005; e-mail SKA@nrc.gov.

Regulatory guides are available for inspection at the Commission's Public Document Room, 2120 L Street NW., Washington, DC. Requests for single copies of draft or final guides (which may be reproduced) or for placement on an automatic distribution list for single copies of future draft guides in specific divisions should be made in writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555. Attention: Distribution and Mail Services Section; or by fax at (301) 415-2260. Telephone requests cannot be

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(5 U.S.C. 552(a))

Dated at Rockville, Maryland, this 7th day of October 1996.

For the Nuclear Regulatory Commission.
Lawrence C. Shao,
*Director, Division of Engineering Technology,
Office of Nuclear Regulatory Research.*
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POSTAL SERVICE

Specifications for Information Based Indicia Program "Host Systems"

AGENCY: Postal Service.

ACTION: Notice of proposed IBIP Host System specification with request for comments and announcement of IBIP Technology Symposium.

SUMMARY: Historically, postage meters have been mechanical and electromechanical devices that (1) maintain through mechanical or electronic "registers" (postal security devices) an account of all postage printed and the remaining balance of prepaid postage, and (2) print postage indicia that are accepted by the Postal Service as evidence of the prepayment of postage. Two proposed specifications have been published on these subjects, and are entitled "Information Based Indicia Program (IBIP) Postal Security Device (PSD) Specification [Draft]" and "Information Based Indicia Program (IBIP) Indicum Specification [Draft]." (See Federal Register notices Vol. 61, No. 128, Tuesday, July 2, 1996, and Vol. 61, No. 136, Monday, July 15, 1996.) The proposed IBIP Host System specification is another specification document intended to provide technical information relevant to the development and production of IBI products. The U.S. Postal Service is seeking comments on this specification.

The Postal Service also seeks comments on intellectual property issues raised by the IBIP Host System specification if adopted in present form. If an intellectual property issue includes patents or patent applications covering any implementations of the specification, the comments should include a listing of such patents and applications and the license terms available for such patents and applications.

Also, the Postal Service will be hosting a technology symposium at the Booz-Allen & Hamilton conference