

SUMMARY: This notice establishes the date when the application period for New Lands Grazing Permits will close. This action is necessary to comply with 25 CFR 700.709.

FOR FURTHER INFORMATION CONTACT: Paul Tessler (Legal Counsel), Office of Navajo and Hopi Indian Relocation, at (520) 779-2727.

SUPPLEMENTARY INFORMATION: On June 9, 1992, the Office of Navajo and Hopi Indian Relocation (ONHIR) published in the Federal Register (Vol. 57, No. 111, at p. 24363) a final rule regarding New Lands Grazing privileges. The rule, 25 CFR 700.709(d) provided that the ONHIR will determine when the application period for New Lands Grazing Permits will close and that a notice of that date would be published.

The ONHIR has determined that pursuant to 25 CFR 700.709, persons on the list of permittees eligible to receive grazing permits must file an application for a New Lands Grazing Permit by June 2, 1997, or they will lose their priority status for receiving permits.

The ONHIR also intends to notify each of the approximately 65 persons eligible to receive a New Lands Grazing Permit by writing to them personally.

Dated: March 11, 1997.

Christopher J. Bavasi,

Executive Director, Office of Navajo and Hopi Indian Relocation.

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NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Proposed Collection; Comment request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of pending NRC action to submit an information collection request to OMB and solicitation of public comment.

SUMMARY: The NRC is preparing a submittal to OMB for review of continued approval of information collections under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35).

Information pertaining to the requirement to be submitted:

1. The title of the information collection: 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities".

2. Current OMB approval number: 3150-0011.

3. How often the collection is required: As necessary in order for NRC

to meet its responsibilities to conduct a detailed review of applications for licenses and amendments thereto to construct and operate nuclear power plants, preliminary or final design approvals, design certifications, research and test facilities, reprocessing plants and other utilization and production facilities, licensed pursuant to the Atomic Energy Act of 1954, as amended (the Act) and to monitor their activities.

4. Who is required or asked to report: Licensees and applicants for nuclear power plants and non-power reactors (research and test facilities).

5. The number of annual respondents: 154.

6. The number of hours needed annually to complete the requirement or request: 5.5M (approximately 2.8M reporting hours and 2.6M recordkeeping hours); an average of 35.6K per respondent.

7. Abstract: 10 CFR Part 50 of the NRC's regulations, "Domestic Licensing of Production and Utilization Facilities," specifies technical information and data to be provided to the NRC or maintained by applicants and licensees so that the NRC may make determinations necessary to promote the health and safety of the public, in accordance with the Act. The reporting and recordkeeping requirements contained in 10 CFR Part 50 are mandatory for the affected licensees and applicants.

Submit, by May 19, 1997, comments that address the following questions:

1. Is the proposed collection of information necessary for the NRC to properly perform its functions? Does the information have practical utility?

2. Is the burden estimate accurate?

3. Is there a way to enhance the quality, utility, and clarity of the information to be collected?

4. How can the burden of the information collection be minimized, including the use of automated collection techniques or other forms of information technology?

A copy of the draft supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street NW (lower level), Washington, DC. Members of the public who are in the Washington, DC, area can access this document via modem on the Public Document Room Bulletin Board (NRC's Advanced Copy Document Library), NRC subsystem at FedWorld, 703-321-3339. Members of the public who are located outside of the Washington, DC, area can dial FedWorld, 1-800-303-9672, or use the FedWorld Internet address: fedworld.gov (Telnet). The document

will be available on the bulletin board for 30 days after the signature date of this notice. If assistance is needed in accessing the document, please contact the FedWorld help desk at 703-487-4608. Additional assistance in locating the document is available from the NRC Public Document Room, nationally at 1-800-397-4209, or within the Washington, DC, area at 202-634-3273.

Comments and questions about the information collection requirements may be directed to the NRC Clearance Officer, Brenda Jo. Shelton, U.S. Nuclear Regulatory Commission, T-6 F33, Washington, DC, 20555-0001, by telephone at (301) 415-7233, or by Internet electronic mail at BJS1@NRC.GOV.

Dated at Rockville, Maryland, this 13th day of March 1997.

For the Nuclear Regulatory Commission.
Gerald F. Cranford,

Designated Senior Official for Information Resources Management.

[FR Doc. 97-7059 Filed 3-19-97; 8:45 am]

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[Docket Nos. STN 50-454, STN 50-455, STN 50-456 and 50-457]

Commonwealth Edison Company; Byron Station, Units 1 and 2; Braidwood Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-37, NPF-66, NPF-72 and NPF-77, issued to Commonwealth Edison Company (ComEd, the licensee), for operation of Byron Station, Units 1 and 2, located in Ogle County, Illinois and Braidwood Station, Units 1 and 2, located in Will County, Illinois.

Environmental Assessment

Identification of the Proposed Action

The proposed action would revise the technical specifications (TS) to take credit for soluble boron in the spent fuel pool in maintaining an acceptable margin of subcriticality. The proposed change would remain in effect until December 31, 1997, at which time the licensee is expected to implement long-term corrective actions.

The Need for the Proposed Action

The proposed action is required in order for the licensee to be in compliance with its TS. Heretofore, the compliance with the requirement to maintain criticality (k_{eff}) in the spent fuel pool to less than 0.95 with

unborated water was accomplished through the use of Boraflex, a neutron absorber. However, recent tests have indicated that the Boraflex is showing degradation induced by gamma radiation. Maintaining boron concentration of 2000 parts per million in the spent fuel pool is more than sufficient to ensure that the k_{eff} is maintained below 0.95.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that the licensee's proposal to take credit for soluble boron in the spent fuel pool water to maintain k_{eff} less than or equal to 0.95 is acceptable.

The change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential nonradiological impacts, the proposed action involves features located entirely within the restricted area as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Accordingly, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternatives with equal or greater environmental impact need not be evaluated. As an alternative to the proposed action, the staff considered denial of the proposed action. Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Byron Station, Units 1 and 2, and Braidwood Station, Units 1 and 2.

Agencies and Persons Consulted

In accordance with its stated policy, on February 11, 1997, the staff

consulted with Frank Niziolek of the Illinois Department of Nuclear Safety, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated November 5, 1996, which is available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document rooms located at: for Byron, the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010; for Braidwood, the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481.

Dated at Rockville, Maryland, this 13th day of March 1997.

For the Nuclear Regulatory Commission.
Robert A. Capra,
Director, Project Directorate III-2, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 97-7060 Filed 3-19-97; 8:45 am]

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Advisory Committee on Reactor Safeguards; Meeting Notice

In accordance with the purposes of Sections 29 and 182b. of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards will hold a meeting on April 3-5, 1997, in Conference Room T-2B3, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the Federal Register on Thursday, January 23, 1997 (62 FR 3539).

Thursday, April 3, 1997

8:30 A.M.-8:45 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting and comment briefly regarding items of current interest. During this session, the Committee will discuss priorities for preparation of ACRS reports.

8:45 A.M.-9:45 A.M.: Proposed Regulatory Approach Associated with Steam Generator Integrity (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed regulatory approach for dealing with steam generator integrity issues.

Representatives of the nuclear industry will participate, as appropriate.

10:00 A.M.-11:30 A.M.: Consequences of Reactor Water Cleanup System Line Break Outside Containment (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the results of the study performed by the staff on the consequences of reactor water cleanup system line break outside containment.

Representatives of the nuclear industry will participate, as appropriate.

11:30 A.M.-11:45 A.M.: Subcommittee Report (Open)—The Committee will hear a report by the Chairman of the Thermal-Hydraulic Phenomena Subcommittee regarding the items discussed during the March 28, 1997 subcommittee meeting.

11:45 A.M.-12:00 Noon: Reconciliation of ACRS Comments and Recommendations (Open)—The Committee will discuss responses from the NRC Executive Director for Operations (EDO) to comments and recommendations included in recent ACRS reports. The EDO responses are expected to be provided to the ACRS prior to the meeting.

1:00 P.M.-2:30 P.M.: Proposed Regulatory Guidance Related to Implementation of 10 CFR 50.59 Requirements (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the proposed regulatory guidance for assessing the adequacy of the licensee's process for implementing the requirements of 10 CFR 50.59, "Changes, Tests and Experiments."

Representatives of the nuclear industry will participate, as appropriate.

2:45 P.M.-6:30 P.M.: Preparation of ACRS Reports (Open)—The Committee will discuss proposed ACRS reports on matters considered during this meeting, as well as proposed reports considered during previous meetings on issues such as shutdown operations risk and plant-specific safety goals.

Friday, April 4, 1997

8:30 A.M.-8:35 A.M.: Opening Remarks by the ACRS Chairman (Open)—The ACRS Chairman will make opening remarks regarding conduct of the meeting.

8:35 A.M.-10:00 A.M.: Boraflex Degradation in Spent Fuel Pool Storage Racks (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the resolution of issues associated with the degradation of Boraflex used in spent fuel pool storage racks and licensee responses to Generic Letter 96-04, "Boraflex Degradation in Spent Fuel Storage Racks."

Representatives of the nuclear industry will participate, as appropriate.

10:15 A.M.-11:45 A.M.: Use of Potassium Iodide After a Severe Accident (Open)—The Committee will hear presentations by and hold discussions with the representatives of the NRC staff regarding the NRC policy on the use of potassium iodide after a severe accident and other related issues.

1:15 P.M.-1:45 P.M.: Future ACRS Activities (Open)—The Committee will discuss the recommendations of the Planning