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Volume VII

None

General Wage Determination Publication

General wage determinations issued under the Davis-Bacon and related Acts, including those noted above, may be found in the Government Printing Office (GPO) document entitled "General Wage Determinations Issued Under The Davis-Bacon and Related Acts". This publication is available at each of the 50 Regional Government Depository Libraries and many of the 1,400 Government Depository Libraries across the country.

The general wage determinations issued under the Davis-Bacon and related Acts are available electronically by subscription to the FedWorld Bulletin Board System of the National Technical Information Service (NTIS) of the U.S. Department of Commerce at (703) 487-4630.

Hard-copy subscriptions may be purchased from: Superintendent of Documents, U.S. Government Printing Office, Washington, D.C. 20402, (202) 512-1800.

When ordering hard-copy subscription(s), be sure to specify the State(s) of interest, since subscriptions may be ordered for any or all of the seven separate volumes, arranged by State. Subscriptions include an annual

edition (issued in January or February) which includes all current general wage determinations for the States covered by each volume. Throughout the remainder of the year, regular weekly updates are distributed to subscribers.

Signed at Washington, D.C. this 25th day of April 1997.

Carl Poleskey,

Chief, Branch of Construction Wage Determinations.

[FR Doc. 97-11112 Filed 4-29-97; 8:45 am]

BILLING CODE 4510-27-M

NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (97-052)]

NASA Advisory Council; Meeting

AGENCY: National Aeronautics and Space Administration.

ACTION: Notice of meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Public Law 92-463, as amended, the National Aeronautics and Space Administration announces a meeting of the NASA Advisory Council.

DATES: May 21, 1997, 8:30 a.m. to 3 p.m.; and May 22, 1997, 8:30 a.m. to noon.

ADDRESSES: Johnson Space Center, Building 1, Room 360, Houston, TX 77058.

FOR FURTHER INFORMATION CONTACT: Ms. Anne L. Accola, Code Z, National Aeronautics and Space Administration, Washington, DC 20546, 202/358-2096.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the seating capacity of the room. The agenda for the meeting is as follows:

- Update on Activities at NASA
- Integrated Financial Management Project
- Full Cost Initiative
- Committee and Task Force Reports
- Technology Development in the HEDS Strategic Enterprise
- Institutional and Programmatic Changes and Challenges at JSC
- Discussion of Findings and Recommendations

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register. Unbadged visitors will be required to obtain a visitor's badge at the Johnson Space Center badging office in Building 110.

Dated: April 24, 1997.

Leslie M. Nolan,

Advisory Committee Managements Officer, National Aeronautics and Space Administration.

[FR Doc. 97-11202 Filed 4-29-97; 8:45 am]

BILLING CODE 7510-01-M

NATIONAL COMMUNICATIONS SYSTEMS**Notice**

AGENCY: National Communications System (NCS).

ACTION: Notice of publication.

SUMMARY: Federal Telecommunication Recommendation (FTR) 1024A-1997, "Project 25 Radio Equipment" was approved for publication on April 16, 1997. This recommendation describes a family of 12.5 kHz bandwidth or less digital radios that can be used in a wide variety of Government applications; and supersedes FTR 1024-1997, dated January 21, 1997. Project 25 is a joint effort of US Federal, state, and local government, and industry. State government participation is through the National Association of State Telecommunications Directors (NASTD); local government participation is through the Association of Public-safety Communications Officials, International (APCO); and industry participation is through the Telecommunications Industry Association (TIA). A copy of FTR 1024A-1997, in PDF format, is available on the World-Wide Web at <<http://members.aol.com/Project25/>>.

FOR FURTHER INFORMATION: Contact Mr. Robert Fenichel at telephone (703) 607-6190, e-mail <fenicher@ncs.gov>, or write to the National Communications System, Attn: N6, 701 South Court House Road, Arlington, VA 22204-2198.

Dennis Bodson,

Chief, Technology and Standards Division.

[FR Doc. 97-11127 Filed 4-29-97; 8:45 am]

BILLING CODE 5000-03-M

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-261]

Carolina Power & Light Company; Notice of Partial Withdrawal of Application for Amendment to Facility Operating License DPR-23

The U.S. Nuclear Regulatory Commission (the Commission) has granted the request of the Carolina Power & Light Company (CP&L or the

licensee) to withdraw the remaining portion of its January 30, 1996, application for proposed amendment to Facility Operating License No. DPR-23 for the H. B. Robinson Steam Electric Plant, Unit No. 2, (HBR) located in Darlington County, South Carolina.

The proposed amendment would have revised the technical specifications (TS) to change the wording of TS 4.6.1.3 to require inspection of the EDGs "at least once every refueling outage" instead of "at each refueling."

The Commission had previously issued a Notice of Consideration of Issuance of Amendment published in the **Federal Register** on February 28, 1996, (61 FR 7546). However, by letter dated March 14, 1997, the licensee withdrew the remaining proposed change.

For further details with respect to this action, see the application for amendment dated January 30, 1996, as supplemented May 20, 1996, and the licensee's letter dated March 14, 1997, which withdrew the remaining portion of the application for license amendment. The above documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Hartsville Memorial Library, 147 West College Avenue, Hartsville, South Carolina 29550.

Dated at Rockville, Maryland, this 23rd day of April 1997.

For the Nuclear Regulatory Commission.

Brenda L. Mozafari,

Project Manager, Project Directorate, Division of Reactor Projects, Office of Nuclear Reactor Regulation.

[FR Doc. 97-11122 Filed 4-29-97; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-254 and 50-265]

Commonwealth Edison company and Midamerican Energy Company; Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. DPR-29 and DPR-30, issued to Commonwealth Edison Company (ComEd, the licensee), for operation of the Quad Cities Nuclear Power Station, Units 1 and 2, located in Rock Island County, Illinois.

The proposed amendments would reflect a change in the Quad Cities, Unit 2, Minimum Critical Power Ratio (MCPR) Safety Limit and add the Siemens Power Corporation (SPC) methodology for application of the Advanced Nuclear Fuel for Boiling Water Reactors (ANFB) Critical Power Correlation to coresident General Electric fuel for Quad Cities, Unit 2, Cycle 15, to Technical Specification (TS) Section 6.9.A.6.b.

This request for amendments was submitted under exigent circumstances to support Quad Cities, Unit 2, Cycle 15, operation which is scheduled to be on line May 19, 1997. On March 20, 1997, SPC determined the need for a larger data base for determining the additive constant uncertainty. The combined time necessary for SPC to develop the new data base and the time for ComEd to develop this TS request would not allow the normal 30-day period for public comment to support Quad Cities, Unit 2, startup.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6), for amendments to be granted under exigent circumstances, the NRC staff must determine that the requested amendments involve no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated:

The probability of an evaluated accident is derived from the probabilities of the individual precursors to that accident. The consequences of an evaluated accident are determined by the operability of plant systems designed to mitigate those consequences. Limits have been established consistent with NRC approved methods to ensure that fuel performance during normal, transient, and accident conditions is acceptable. The proposed Technical Specifications amendment conservatively establishes the MCPR Safety Limit for Quad Cities Unit 2, such that the fuel is protected

during normal operation and during any plant transients or anticipated operational occurrences. Additionally, methodologies are being added to the Section 6.9.A.6.b list of methodologies utilized in determining core operating limits.

a. MCPR Safety Limit and MCPR Safety Limit Bases Change

The probability of an evaluated accident is not increased by increasing the MCPR Safety Limit to 1.10 and changing the MCPR Safety Limit Bases. The change does not require any physical plant modifications, physically affect any plant components, or entail changes in plant operation. Therefore, no individual precursors of an accident are affected.

This Technical Specification amendment proposes to change the MCPR Safety Limit to protect the fuel during normal operation as well as during any transients or anticipated operational occurrences. The method that is used to determine the ATRIUM-9B additive constant uncertainty is conservative, such that, the resulting MCPR Safety Limit is high enough to ensure that less than 0.1% of the fuel rods are expected to experience boiling transition if the limit is not violated. Operational limits will be established based on the proposed MCPR Safety Limit to ensure that the MCPR Safety Limit is not violated during all modes of operation. This will ensure that the fuel design safety criteria, more than 99.9% of the fuel rods avoiding transition boiling during normal operation as well as anticipated operational occurrences, is met. The method for calculating an ATRIUM-9B additive constant uncertainty, is described in Reference 2 [SPC document, *ANFB Critical Power Correlation Uncertainty For Limited Data Sets*, ANF-1125(P), Supplement 1, Appendix D, Siemens Power Corporation—Nuclear Division, Submitted on April 18, 1997] and is based on an expanded pool of data for the ATRIUM-9B fuel design (527 data points). The additive constant uncertainty from Reference 2 is then used to determine the change from the additive constant uncertainty using the original pool of data (125 data points). This difference is conservatively doubled and added to the additive constant uncertainty using the original pool of data (125 data points). Reference 5 [Siemens Power Corporation letter, "Interim Use of Increased ANFB Additive Constant Uncertainty", HDC:97:033, H.D. Curet to Document Control Desk, April 18, 1997] documents the conservative interim approach of doubling the difference in additive constant uncertainties. The resulting additive constant uncertainty is used to determine the Quad Cities Unit 2 Cycle 15 MCPR Safety Limit. Since the new MCPR Safety Limit was determined using a conservative ATRIUM-9B additive constant uncertainty, and the operability of plant systems designed to mitigate any consequences of accidents have not changed, the consequences of an accident previously evaluated are not expected to increase.

b. Addition of Siemens Power Corporation's (SPC) methodology for Application of the ANFB Critical Power Correlation to Coresident GE Fuel for Quad Cities Unit 2 Cycle 15 to Section 6.9.A.6.b