

displays a currently valid OMB control number.

1. Type of submission, new, revision, or extension: Revision

2. The title of the information collection: Final rule, Requirements for certain generally licensed industrial devices containing byproduct material (10 CFR Parts 30, 31, and 32)

3. The form number, if applicable: NRC Form 653, Transfers of Industrial Devices Report

4. How often the collection is required: Quarterly, annually, on occasion

5. Who is required or asked to report: NRC licensees and Agreement State licensees

6. An estimate of the number of responses:

Part 30: 5 responses (5 NRC licensees);

Part 31: 65,904 responses (13,251 NRC licensees and 52,653 Agreement State licensees);

Part 32: 16,278 responses (4,277 NRC licensees and 12,001 Agreement State licensees);

NRC Form 653: 2,208 responses (504 NRC licensees and 1,704 Agreement State licensees)

7. The estimated number of annual respondents:

Part 30: 5 respondents (5 NRC licensees);

Part 31—30,380 respondents (7,595 NRC licensees and 22,785 Agreement State licensees);

Part 32—92 respondents (21 NRC licensees and 71 Agreement State licensees);

NRC Form 653—92 respondents (21 NRC licensees and 71 Agreement State licensees).

8. An estimate of the number of hours needed annually to complete the requirement or request:

Part 30—3 hours (NRC licensees);

Part 31—35,488 hours (35,792 reporting hours—7,884 hours NRC licensees and 27,907 hours Agreement State licensees and a reduction of 304 recordkeeping hours—76 hours NRC licensees and 228 hours Agreement State licensees).

Part 32—583 hours (490 reporting hours—128 hours NRC licensees and 362 hours Agreement State licensees plus 93 recordkeeping hours—21 hours NRC licensees and 72 hours Agreement State licensees).

Form 653—516 hours (118 hours NRC licensees and 398 hours Agreement State licensees).

9. An indication of whether Section 3507(d), Pub. L. 104-13 applies: Applicable

10. Abstract: The final rule amends regulations governing the use of

byproduct material in certain measuring, gauging, or controlling devices. The amendments include specific criteria for inclusion in the registration program and details about the information required. The amendments also modify the quarterly transfer reporting, recordkeeping, and labeling requirements for specific licensees who distribute these generally licensed devices and provide clarification concerning provisions of the regulations applicable to all general licensees for byproduct material. The rule is intended to allow the NRC to better track general licensees so that they can be contacted or inspected, to make sure that the devices can be identified even if lost or damaged, and to further ensure that general licensees are aware of and understand the requirements for the possession of devices containing byproduct material. Greater awareness helps to ensure that general licensees will comply with the requirements for proper handling and disposal of generally licensed devices and would reduce the potential for incidents that could result in unnecessary radiation exposure to the public and contamination of property.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (<http://www.nrc.gov/NRC/PUBLIC/OMB/index.html>). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer by September 22, 2000: Erik Godwin Office of Information and Regulatory Affairs (3150-0017, -0016, and -0001), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395-3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301-415-7233.

Dated at Rockville, Maryland, this 16th day of August, 2000.

For the Nuclear Regulatory Commission.

Brenda Jo. Shelton,

NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 00-21514 Filed 8-22-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

Agency Information Collection Activities: Submission for OMB Review; Comment Request

AGENCY: U.S. Nuclear Regulatory Commission (NRC).

ACTION: Notice of the OMB review of information collection and solicitation of public comment.

SUMMARY: The NRC has recently submitted to OMB for review the following proposal for the collection of information under the provisions of the Paperwork Reduction Act of 1995 (44 U.S.C. Chapter 35). The NRC hereby informs potential respondents that an agency may not conduct or sponsor, and that a person is not required to respond to, a collection of information unless it displays a currently valid OMB control number.

Information pertaining to the requirement to be submitted:

1. Type of submission, new, revision, or extension: Revision to the extension.

2. The title of the information collection: 10 CFR Part 71, "Packaging and Transportation of Radioactive Material."

3. The form number, if applicable: None.

4. How often the collection is required: Applications for package certification may be made at any time. Required reports are collected and evaluated on a continuing basis as events occur.

5. Who is required or asked to report: All NRC specific licensees who place byproduct, source, or special nuclear material into transportation, and all persons who wish to apply for NRC approval of package designs for use in such transportation.

6. An estimate of the number of responses: 1,106 responses.

7. The number of annual respondents: 350 licensees.

8. The number of hours needed annually to complete the requirement or request: 57,012 hours for reporting requirements and 6,825 for recordkeeping requirements, or a total of 63,837 hours (approximately 182 hours per respondent).

9. An indication of whether Section 3507(d), Pub. L. 104-13 applies: Not applicable.

10. Abstract: NRC regulations in 10 CFR Part 71 establish requirements for packing, preparation for shipment, and transportation of licensed material, and prescribe procedures, standards, and requirements for approval by NRC of packaging and shipping procedures for

fissile material and for quantities of licensed material in excess of Type A quantities.

A copy of the final supporting statement may be viewed free of charge at the NRC Public Document Room, 2120 L Street, NW (lower level), Washington, DC. OMB clearance requests are available at the NRC worldwide web site (<http://www.nrc.gov/NRC/PUBLIC/OMB/index.html>). The document will be available on the NRC home page site for 60 days after the signature date of this notice.

Comments and questions should be directed to the OMB reviewer by September 22, 2000: Erik Godwin, Office of Information and Regulatory Affairs (3150-0008), NEOB-10202, Office of Management and Budget, Washington, DC 20503.

Comments can also be submitted by telephone at (202) 395-3087.

The NRC Clearance Officer is Brenda Jo. Shelton, 301-415-7233.

Dated at Rockville, Maryland, this 17th day of August 2000.

For the Nuclear Regulatory Commission.

Beth C. St. Mary,

Acting NRC Clearance Officer, Office of the Chief Information Officer.

[FR Doc. 00-21515 Filed 8-22-00; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-412]

Pennsylvania Power Company, Ohio Edison Company, the Cleveland Electric Illuminating Company, the Toledo Edison Company, FirstEnergy Nuclear Operating Company, Beaver Valley Power Station, Unit 2; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-73 issued to FirstEnergy Nuclear Operating Company (the licensee) for operation of the Beaver Valley Power Station, Unit 2 (BVPS-2), located in Beaver County, Pennsylvania.

This notice supersedes the notice published on July 12, 2000 (65 FR 43046) in its entirety.

The proposed amendment would: (1) revise Technical Specification (TS) requirements regarding the minimum number of radiation monitoring

instrumentation channels required to be operable during movement of fuel within the containment; (2) revise the Modes in which the surveillance specified by Table 4.3-3, "Radiation Monitoring Instrumentation Surveillance Requirements," Item 2.c.ii is required; (3) revise TS 3.9.4, "Containment Building Penetrations," to allow both personnel air lock (PAL) doors and other containment penetrations to be open during movement of fuel assemblies within containment, provided certain conditions are met; (4) revise applicability and action statement requirements of TS 3.9.4. to be for only during movement of fuel assemblies within containment; (5) revise periodicity and applicability of Surveillance Requirement (SR) 4.9.4.1; (6) revise SR 4.9.4.2 to verify flow rate of air to the supplemental leak collection and release system (SLCRS) rather than verifying the flow rate through the system; (7) add two new SRs, 4.9.4.3 and 4.9.4.4, for verification and demonstration of SLCRS operability; (8) modify TS 3/4.9.9 for the containment purge exhaust and isolation system to be applicable only during movement of fuel assemblies within containment; (9) revise associated TS Bases as well as make editorial and format changes; and, (10) revise the BVPS-2 Updated Final Safety Analysis Report (UFSAR) description of a fuel-handling accident (FHA) and its radiological consequences. The changes to the BVPS-2 UFSAR reflect a revised FHA analysis that the licensee performed to evaluate the potential consequences of having containment penetrations and/or the PAL open during movement of fuel assemblies within containment. These UFSAR revisions include potential exclusion area boundary, low population zone, and control room operator doses as a result of an FHA.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or

(3) involve a significant reduction in a margin of safety.

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

The proposed amendment involves changes to accident mitigation system requirements. These systems are related to controlling the release of radioactivity to the environment and are not considered to be accident initiators to any previously analyzed accident.

Therefore, the proposed change does not involve a significant increase in the probability of an accident previously evaluated.

Based on the current technical specification requirements, an environmental release due to a fuel handling accident (FHA) occurring within containment is precluded by a design which automatically isolates the containment following detection of radioactivity by redundant containment purge monitors. The proposed amendment, which permits containment penetrations to be open during movement of fuel assemblies within containment, increases the dose at the site boundary and the control room operator dose due to a FHA occurring within containment; however, the dose remains within acceptable limits. Based on a radiological analysis of a FHA within containment with open containment penetrations being filtered by the Supplemental Leak Collection and Release System (SLCRS), the resultant radiological consequences of this event are well within the 10 CFR Part 100.11 limits, as defined by acceptance criteria in the Standard Review Plan (SRP) Section 15.7.4. Control room operator doses remain less than the 10 CFR Part 50 Appendix A General Design Criteria (GDC) 19 limit of 5 rem whole body or its equivalent to any part of the body. The proposed changes to LCO 3.9.4 and associated surveillance requirements will ensure that SLCRS filtration assumptions in the associated radiological analysis are met.

LCO 3.9.10 titled "Water Level—Reactor Vessel" will continue to ensure that at least 23 feet of water is maintained over the fuel during fuel movement when the plant is in Mode 6. LCO 3.9.3 titled "Decay Time" will continue to ensure that irradiated fuel is not moved in the reactor pressure vessel until at least 150 hours after shutdown. These LCOs will continue to ensure that two of the key assumptions used in the radiological safety analysis are met.

The radiological consequences of the Core Alteration events other than the FHA remain unchanged. These events do not result in fuel cladding integrity damage. A radioactive release to the environment is not postulated since the activity is contained in the fuel rods. Therefore, the affected containment systems are not required to mitigate a radioactive release to the environment due to a Core Alteration event.

The proposed revision in the minimum number of the Containment Purge Exhaust