

June 19, 2001.

Beth M. McCormick,

*Advisory Committee Management Officer,
National Aeronautics and Space
Administration.*

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NATIONAL AERONAUTICS AND SPACE ADMINISTRATION

[Notice (01-081)]

NASA Advisory Council (NAC), Space Science Advisory Committee (SScAC), Sun-Earth Connection Advisory Subcommittee

AGENCY: National Aeronautics and
Space Administration.

ACTION: Notice of Meeting.

SUMMARY: In accordance with the Federal Advisory Committee Act, Pub. L. 92-463, as amended, the National Aeronautics and Space Administration announces a forthcoming meeting of the NASA Advisory Council, Space Science Advisory Committee, Sun-Earth Connection Advisory Subcommittee.

DATES: Monday, July 23, 2001, 8:30 a.m. to 6 p.m.; Tuesday, July 24, 2001, 8:30 a.m. to 5 p.m.

ADDRESSES: National Aeronautics and Space Administration, Conference Room 6H46, 300 E Street, SW, Washington, DC, 20546.

FOR FURTHER INFORMATION CONTACT: Dr. George L. Withbroe, Code S, National Aeronautics and Space Administration, Washington, DC 20546, 202/358-2150.

SUPPLEMENTARY INFORMATION: The meeting will be open to the public up to the capacity of the room. The agenda for the meeting includes the following topics:

- State of the Sun-Earth Connection Theme
- Geospace Management Operations Working Group
- Living With a Star Science Architecture Committee
- Solar/Heliospheric Management Operation Working Group
- Report of Discipline Scientists

It is imperative that the meeting be held on these dates to accommodate the scheduling priorities of the key participants. Visitors will be requested to sign a visitor's register.

Dated: June 19, 2001.

Beth M. McCormick,

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-387 and 50-388]

PPL Susquehanna, LLC; Susquehanna Steam Electric Station Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of amendments to Facility Operating License (FOL) Nos. NPF-14, and NPF-22, issued to PPL Susquehanna, LLC (the licensee), for operation of the Susquehanna Steam Electric Station (SSES), Units 1 and 2, located in Luzerne County, Pennsylvania.

Environmental Assessment

Identification of the Proposed Action

The proposed license amendment would revise the FOLs and Technical Specifications (TS) of SSES, Units 1 and 2, to allow the licensee to increase the licensed core power level from 3441 MWt to 3489 MWt, which represents a 1.4 percent increase in the allowable thermal power. SSES Unit 1 was granted conditional authorization for power production by its FOL issued on July 17, 1982. Full power operation of Unit 1 at 3,293 MWt core power was authorized by Amendment No. 5 to the FOL, issued on November 12, 1982. Amendment No. 143 to the FOL, issued on March 22, 1995, authorized a power uprate for Unit 1 to 3,441 MWt. SSES Unit 2 was granted conditional authorization for power production by its FOL issued on March 23, 1984. Full power operation of Unit 2 at 3,293 MWt core power was authorized by Amendment No. 1 to the FOL, issued on June 27, 1984. Amendment No. 103 to the FOL, issued on April 11, 1994, authorized a power uprate for Unit 2 to 3,441 MWt.

The proposed action is in accordance with the licensee's application for license amendment dated October 30, 2000, as supplemented by letters dated February 5, May 22, and May 31, 2001.

The Need for the Proposed Action

The proposed action would allow an increase in power generation at SSES, Units 1 and 2, to provide additional electrical power for distribution to the grid. Power uprate has been widely recognized by the industry as a safe and cost-effective method to increase generating capacity.

Environmental Impacts of the Proposed Action

The environmental impact associated with operation of SSES, Units 1 and 2,

has been previously evaluated by the U.S. Atomic Energy Commission in the "Final Environmental Statement Related to Operation of Susquehanna Steam Electric Station, Units 1 and 2," dated June 1981. In this evaluation, the staff considered the potential doses due to postulated accidents for the site, at the site boundary, and to the population within 50 miles of the site. With regard to consequences of postulated accidents, the licensee has reevaluated the current design basis accidents (DBAs) in its application for license amendments and determined that accident source terms are based on core power levels that bound the proposed core power level of 3489 MWt. Therefore, the current analyses bound the potential doses due to DBAs based on the proposed 1.4 percent increased core power level. No increase in the probability of these accidents is expected to occur.

With regard to normal releases, the licensee has calculated the potential impact on the radiological effluents from the proposed 1.4 percent increase in power level. The licensee concluded that the offsite doses from normal effluent releases remain significantly below the bounding limits of Title 10 of the Code of Federal Regulations (10 CFR), Part 50, Appendix I. Normal annual average gaseous releases remain limited to a small fraction of 10 CFR Part 20, Appendix B, Table 2 limits. The licensee evaluated the effects of power uprate on the radiation sources within the plant and the radiation levels during normal operating conditions. Post-operation radiation levels are expected to increase slightly due to the power uprate; but are expected to have no significant effect on the plant. Occupational doses for normal operations will be maintained within acceptable limits by the site ALARA (as-low-as-reasonably-achievable) program. Solid and liquid waste production may increase slightly as a result of the proposed 1.4 percent uprate; however, waste processing systems are expected to operate within their design requirements.

The NRC has completed its evaluation of the proposed action and concludes that the proposed action will not increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released offsite, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not involve any historic

sites. With regard to thermal discharges to the Susquehanna River, the staff has previously evaluated temperature effects during normal operations at full power and determined the temperature impact on the river to be insignificant. The licensee indicated that an increase in the cooling tower air flow rate will compensate for the slight increase in condenser outlet circulating water temperature, such that no perceptible change in the temperature of the cooling tower basin blowdown to the Susquehanna River is expected. Therefore, the temperature effects on the river will be insignificant. Existing administrative controls ensure the conduct of adequate monitoring such that appropriate actions can be taken to preclude exceeding the limits imposed by the National Pollution Discharge Elimination System permit. No additional requirements or other changes are required as a result of the power uprate. No other non-radiological impacts are associated with the proposed action.

Based upon the above, the NRC concludes that the proposed action does not affect non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the SSES, Units 1 and 2.

Agencies and Persons Consulted

In accordance with its stated policy, on June 19, 2001, the staff consulted with the Pennsylvania State official, Mr. Michael Murphy of the Pennsylvania Department of Environmental Protection, regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

On the basis of the environmental assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated October 30, 2000, as supplemented by letters dated February 5, May 22, and May 31, 2001. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the Internet at the NRC web site, <http://www.nrc.gov/NRC/ADAMS/index.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 19th day of June 2001.

For the Nuclear Regulatory Commission.

Richard P. Correia,

Acting Chief, Section 1, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 01-15815 Filed 6-22-01; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-354]

PSEG Nuclear LLC; Hope Creek Generating Station Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from certain requirements of Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Appendix G, for Facility Operating License No. NPF-57, issued to PSEG Nuclear LLC, (the licensee) for operation of the Hope Creek Generating Station (HCGS), located in Salem County, New Jersey.

Environmental Assessment

Identification of the Proposed Action

Title 10 of the Code of Federal Regulations, Part 50, Appendix G,

requires that pressure-temperature (P-T) limits be established for reactor pressure vessels (RPVs) during normal operating and hydrostatic or leak rate testing conditions. Specifically, 10 CFR Part 50, Appendix G, states, "The appropriate requirements on both the pressure-temperature limits and the minimum permissible temperature must be met for all conditions." The purpose of 10 CFR Part 50, Appendix G, is to protect the integrity of the reactor coolant pressure boundary in nuclear power plants. This is accomplished through these regulations that, in part, specify fracture toughness requirements for ferritic materials of the reactor coolant pressure boundary. Appendix G of 10 CFR Part 50 specifies that the requirements for these limits are the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Appendix G Limits.

The proposed action would exempt HCGS from application of specific requirements of 10 CFR Part 50, Appendix G, and would substitute use of ASME Code Cases N-588 and N-640 as alternatives pursuant to 10 CFR 50.60(b).

The proposed action is in accordance with the licensee's application for exemption dated December 1, 2000, as supplemented by letters dated February 12, May 7, and May 14, 2001.

The Need for the Proposed Action

The proposed action is needed to allow the licensee to implement ASME Code Cases N-588 and N-640 in order to revise the method used to determine the P-T limits.

Code Case N-588, "Alternative to Reference Flaw Orientation of Appendix G for Circumferential Welds in Reactor Vessels, Section XI, Division 1," amends the provisions of the 1989 Edition of ASME Section XI, Appendix G, by permitting the postulation of a circumferentially oriented reference flaw as the limiting flaw in a RPV circumferential weld for the purpose of establishing RPV P-T limits. The 1989 Edition of ASME Section XI, Appendix G, would require that such a reference flaw be postulated as an axially oriented flaw in the circumferential weld. The licensee addressed the technical justification for this exemption by citing industry experience and aspects of RPV fabrication which support the postulation of circumferentially oriented flaws for these welds. The reference flaw is a postulated flaw that accounts for the possibility of a prior existing defect that may have gone undetected during the fabrication process. Postulating the Appendix G reference flaw in a circumferential weld