

200 Exelon Way, Kennett Square, PA 19348.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions, and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer, or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)–(v) and 2.714(d).

Detailed information about the license renewal process can be found under the Nuclear Reactors icon of the NRC's Web page at <http://www.nrc.gov>.

A copy of the application to renew the operating licenses for PBAPS Units 2 and 3 is available for public inspection at the Commission's Public Document Room, 11555 Rockville Pike (first floor), Rockville, Maryland, 20855–2738, and on the NRC's Web page at <http://www.nrc.gov>. The NRC maintains an Agencywide Documents Access and Management System (ADAMS), which provides text and image files of NRC's public documents. These documents may be accessed through the NRC's Public Electronic Reading Room on the Internet at <http://www.nrc.gov/NRC/ADAMS/index.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1–800–397–4209, 301–415–4737 or by e-mail to [pdr@nrc.gov](mailto:pdr@nrc.gov).

The staff has also verified that copies of the license renewal application for the PBAPS, Units 2 and 3 are also available to local residents at the Harford County Public Library, in Whiteford, Maryland, and the Collinsville Community Library, in Brogue, Pennsylvania.

Dated at Rockville, Maryland, this the 24th day of August 2001.

For the Nuclear Regulatory Commission.

**Christopher I. Grimes,**

*Chief, License Renewal and Standardization Branch, Division of Regulatory Improvement Programs, Office of Nuclear Reactor Regulation.*

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**BILLING CODE 7590–01–P**

## **NUCLEAR REGULATORY COMMISSION**

**[Docket No. 50–313]**

### **Entergy Operations, Inc.; Arkansas Nuclear One, Unit 1, Environmental Assessment and Finding of No Significant Impact**

The U.S. Nuclear Regulatory Commission (NRC) is considering the issuance of an amendment to Renewed Facility Operating License No. DPR–51, issued to Entergy Operations, Inc. (the licensee), for operation of Arkansas Nuclear One, Unit 1, (ANO–1) located in Pope County, Arkansas.

#### **Environmental Assessment**

##### *Identification of the Proposed Action*

The proposed amendment would revise the existing, or current, Technical Specifications (CTS) for ANO–1 in their entirety, based on the guidance provided in NUREG–1430, “Standard Technical Specifications, Babcock and Wilcox Plants,” and in the NRC's regulations, including 10 CFR 50.36, “Technical specifications.”

The proposed amendment is in accordance with the licensee's application dated January 28, 2000, as supplemented by letters dated August 9 and September 28, 2000, and February 6, March 19, May 1, and August 23, 2001.

##### *The Need for the Proposed Action*

It has been recognized that nuclear safety in all nuclear power plants would benefit from an improvement and standardization of plant Technical Specifications (TS). The “Interim Policy Statement on Technical Specification Improvements for Nuclear Power Plants,” (52 FR 3788) contained proposed criteria for defining the scope of TS. Later, the NRC's “Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors,” published on July 22, 1993 (58 FR 39132), incorporated lessons learned since publication of the interim policy statement and formed the basis for revisions to 10 CFR 50.36. In 1995, the NRC published a Final Rule amending 10 CFR 50.36 (60 FR 36953) in which the NRC codified criteria for determining the content of TS. To facilitate the development of standard TS for nuclear power reactors, each power reactor vendor owners' group (OG) and the NRC staff developed standard TS. For ANO–1, the Improved Standard Technical Specifications (ISTS) are in NUREG–1430. This document forms part of the basis for the

proposed ANO–1 Improved Technical Specifications (ITS) conversion.

##### *Description of the Proposed Change*

The proposed changes to the CTS are based on NUREG–1430 and on guidance provided by the NRC in its Final Policy Statement and subsequent revision to 10 CFR 50.36. The objective of the changes is to completely rewrite, reformat, and streamline the CTS (i.e., to convert the CTS to the ITS). Emphasis is placed on human factors principles to improve clarity and understanding of the TS. The Bases section of the ITS has been significantly expanded to clarify and better explain the purpose and foundation of each specification. In addition to NUREG–1430, portions of the CTS were also used as the basis for the development of the ANO–1 ITS.

The licensee has categorized the proposed changes to the CTS into four general groupings. These groupings are characterized as administrative changes, relocation changes, more restrictive changes, and less restrictive changes.

Administrative changes are those that involve restructuring, renumbering, rewording, interpretation, and complex rearranging of requirements and other changes not affecting technical content or substantially revising an operating requirement. The reformatting, renumbering, and rewording process reflects the attributes of NUREG–1430 and does not involve technical changes to the existing TS. The proposed changes include: (a) providing the appropriate numbers, etc., for NUREG–1430 bracketed information (information that must be supplied on a plant-specific basis, and which may change from plant to plant); (b) identifying plant-specific wording for system names, etc.; and (c) changing NUREG–1430 section wording to conform to existing licensee practices. Such changes are administrative in nature and do not impact initiators of analyzed events or assumed mitigation of accident or transient events.

Relocation changes are those involving relocation of requirements and surveillances for structures, systems, components, or variables that do not meet the criteria for inclusion in TS. Current TS requirements that do not satisfy or fall within any of the four criteria specified in the NRC's policy statement may be relocated to appropriate licensee-controlled documents. The requirements and surveillances for these affected structures, systems, components, or variables would be relocated from the TS to administratively controlled documents such as the quality assurance program, the Final Safety

Analysis Report, the ITS BASES, the Technical Requirements Manual, the Core Operating Limits Report (COLR), the Offsite Dose Calculation Manual (ODCM), the Inservice Testing Program, or other licensee-controlled documents. Changes made to these documents would be made pursuant to 10 CFR 50.59, "Changes, tests, and experiments," or other appropriate control mechanisms, and may, within the prescribed limits, be made without prior NRC review and approval. In addition, the affected structures, systems, components, or variables are often addressed in existing surveillance procedures that are also subject to 10 CFR 50.59. These proposed changes to the TS will not, in and of themselves, impose any requirements.

More restrictive changes are those involving more stringent requirements compared to the CTS for operation of the facility. These more stringent requirements do not result in operation that will alter assumptions relative to the mitigation of an accident or transient event. The more restrictive requirements will not alter the operation of process variables, structures, systems, or components described in the safety analyses. For each requirement in the CTS that is more restrictive than the corresponding requirement in NUREG-1430 that the licensee proposes to retain in the ITS, they have provided an explanation of why they have concluded that retaining the more restrictive requirement is desirable to ensure safe operation of the facility because of specific design features of the plant.

Less restrictive changes are those where CTS requirements are relaxed or eliminated, or new plant operational flexibility is provided. The more significant "less restrictive" requirements are justified on a case-by-case basis. When requirements have been shown to provide little or no safety benefit, their removal from the TS may be appropriate. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of (a) generic NRC actions, (b) new NRC staff positions that have evolved from technological advancements and operating experience, or (c) resolution of the OGs' comments on the ISTS. The licensee's design will be reviewed to determine if the specific design basis and licensing basis are consistent with the technical basis for the model requirements in NUREG-1430, thus providing a basis for these revised TS, or, if relaxation of the requirements in the CTS is warranted, based on the justification provided by the licensee.

These administrative, relocation, more restrictive, and less restrictive changes to the requirements of the CTS do not result in operations that will alter assumptions relative to mitigation of an analyzed accident or transient event.

In addition to the proposed changes solely involving the conversion, there are also changes proposed that are different from the requirements in both the CTS and the ISTS (NUREG-1430). These proposed beyond-scope issues to the ITS conversion are as follows:

1. ITS Limiting Condition for Operation (LCO) 3.2.3, "Axial Power Imbalance Operating Limits"—Completion time for power reduction if axial power imbalance not restored to within limits changed to 4 hours from value in NUREG-1430 (2 hours).

2. ITS LCO 3.2.4, "Quadrant Power Tilt (QPT)"—Revised the completion time for several actions for circumstances where QPT exceeds limits specified in the COLR.

3. ITS LCO 3.4.8, "RCS [Reactor Coolant System] Loops, MODE 5, Loops Not Filled"—Added a required action to suspend operations involving reduction in RCS water volume if required decay heat removal (DHR) loops were not operable or required DHR loop not in operation.

4. ITS LCO 3.4.11, "Low Temperature Overpressure Protection (LTOP) System"—Adopted some of the NUREG-1430 required actions and surveillance requirements which are more restrictive than CTS but did not adopt all NUREG-1430 requirements.

5. ITS LCO 3.5.2, "ECCS [Emergency Core Cooling System]—Operating"—Added a shutdown requirement for a condition where less than 100 percent of the ECCS flow equivalent to a single operable train is available.

6. ITS LCO 3.7.1, "Main Steam Safety Valves (MSSVs)"—Reformatted to replace figure in NUREG-1430 with a table providing limitations for operation with more than one inoperable MSSV per steam generator.

7. ITS LCO 3.4.13, "RCS Operational LEAKAGE"—Modified surveillance requirement to specify that the surveillance is not required until after the plant is at or near operating pressure.

8. ITS Administrative Controls 5.5.1, "Offsite Dose Calculation Manual (ODCM)"—Reference reports by name only instead of NUREG-1430 convention of including report name and associated TS.

9. ITS Administrative Controls 5.2.2, "Unit Staff"—Reference to specific operator staffing requirements is replaced with a reference to the applicable regulation.

10. ITS LCO 3.6.3, "Reactor Building Isolation Valves"—Surveillance requirement in NUREG-1430 not adopted for reactor building purge valves since ANO-1 does not have resilient seated valves.

11. ITS LCO 3.6.4, "Reactor Building Pressure"—Lower limit on reactor building pressure increased to a more restrictive value to be consistent with ECCS analyses and Bases statements in NUREG-1430.

#### *Environmental Impacts of the Proposed Action*

The NRC has completed its evaluation of the proposed conversion of the CTS to the ITS for ANO-1, including the beyond-scope issues discussed above. Changes which are administrative in nature have no effect on the technical content of the TS. The increased clarity and understanding these changes bring to the TS are expected to improve the operators control of ANO-1 in normal and accident conditions.

Relocation of requirements from the CTS to other licensee-controlled documents does not change the substance of these provisions requirements themselves. Future changes to these provisions may then be made by the licensee under 10 CFR 50.59 and other NRC-approved control mechanisms which will ensure continued adequate control of their substance. All such relocations would be consistent with the guidelines of NUREG-1430 and 10 CFR 50.36.

Changes involving more restrictive requirements enhance plant safety.

Changes involving less restrictive requirements have been reviewed individually. When requirements have been shown to provide little or no safety benefit, or to place an unnecessary burden on the licensee, their removal from the TS is justified. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of a generic action, or of agreements reached during discussions with the OG, and found to be acceptable for the plant. Generic relaxations contained in NUREG-1430 have been reviewed by the NRC staff and found to be acceptable.

The proposed amendment will not significantly increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released off site, and there is no significant increase in occupational or public radiation exposure. The changes will not create any new or unreviewed environmental impacts that were not considered in the Final Environmental Statement (FES) related to the operation of ANO-1,

dated February 9, 1973, and the Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Regarding the Arkansas Nuclear One, Unit 1 (NUREG 1437-Supplement 3) published in April 2001. Therefore, there are no significant radiological environmental impacts associated with the proposed amendment.

With regard to potential non-radiological impacts, the proposed amendment does not have a potential to affect any historic sites. It involves features located entirely within the restricted area for the plant defined in 10 CFR Part 20. It does not affect non-radiological plant effluents and has no other environmental impact. It does not increase any discharge limit for the plant. Therefore, there are no significant non-radiological environmental impacts associated with the proposed amendment.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed amendment.

#### *Environmental Impacts of the Alternatives to the Proposed Action*

As an alternative to the proposed action, the staff considered denial of the proposed action (i.e., the "no-action" alternative). Denial of the licensee's application would result in no change in current environmental impacts of ANO-1 operations, but it would prevent the safety benefits to the plant from the conversion to the ITS. The environmental impacts of the proposed action and the alternative action are similar.

#### *Alternative Use of Resources*

This action does not involve the use of any different resources that those previously considered in the FES or Supplement 3 to NUREG-1437 for ANO-1.

#### *Agencies and Persons Consulted*

In accordance with its stated policy, on July 31, 2001, the staff consulted with the Arkansas State official, B. Beville of the Arkansas Department of Health, regarding the environmental impact of the proposed amendment. The State official had no comments.

#### **Finding of No Significant Impact**

Based upon the environmental assessment, the NRC concludes that the proposed amendment will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's application dated January 28, 2000, as supplemented by letters dated August 9 and September 28, 2000, and February 6, March 19, May 1, and August 23, 2001. Documents may be examined, and/or copied for a fee, at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/NRC/ADAMS/index.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr@nrc.gov](mailto:pdr@nrc.gov).

Dated at Rockville, Maryland, this 27th day of August, 2001.

Dated at Rockville, Maryland, this 27th day of August 2001.

For the Nuclear Regulatory Commission.

**Robert A. Gramm,**

*Chief, Section 1, Project Directorate IV, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.*

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**BILLING CODE 7590-01-P**

## **SECURITIES AND EXCHANGE COMMISSION**

**[Release No. 35-27434]**

### **Filings Under the Public Utility Holding Company Act of 1935, as Amended ("Act")**

August 27, 2001.

Notice is hereby given that the following filing(s) has/have been made with the Commission pursuant to provisions of the Act and rules promulgated under the Act. All interested persons are referred to the application(s) and/or declaration(s) for complete statements of the proposed transactions(s) summarized below. The application(s) and/or declaration(s) and any amendment(s) is/are available for public inspection through the Commission's Branch of Public Reference.

Interested persons wishing to comment or request a hearing on the application(s) and/or declaration(s) should submit their views in writing by September 21, 2001, to the Secretary, Securities and Exchange Commission, Washington, DC 20549-0609, and serve a copy on the relevant applicant(s) and/

or declaration(s) at the address(es) specified below. Proof of service (by affidavit or, in the case of an attorney at law, by certificate) should be filed with the request. Any request for hearing should identify specifically the issues of facts or law that are disputed. A person who so requests will be notified of any hearing, if ordered, and will receive a copy of any notice or order is issued in the matter. After September 21, 2001, the application(s) and/or declaration(s), as filed or as amended, may be granted and/or permitted to become effective.

### **Connecticut Light and Power Company (70-9905)**

The Connecticut Light and Power Company ("CL&P"), a wholly owned electric utility subsidiary of Northeast Utilities ("NU"), a public utility holding company, and CL&P Receivables Corporation ("CRC"), a wholly-owned special purpose subsidiary of CL&P, both located at 107 Selden Street, Berlin, Connecticut 06037-5457, have filed a declaration under section 12(c) and rules 46 and 54 of the Act.

By order dated September 29, 1997 (HCAR No. 26761) ("1997 Order"), the Commission authorized CL&P to engage in five transactions in connection with its receivables program ("Program"). Under the 1997 Order, authority was granted for (i) CL&P to organize CRC, (ii) CRC to issue shares of common stock, (iii) CL&P to acquire shares of CRC common stock, (iv) CL&P to make, directly and indirectly, initial and general equity contributions to CRC, and (v) CRC to pay dividends to CL&P from time to time out of capital to achieve the optimum balance of capital to achieve economic efficiency. Transactions (i) through (iv) (with respect to initial equity contributions) have been undertaken and by their nature are permanent, while (v) by its nature is an ongoing process as the Program moves forward. The Program was scheduled to expire on July 11, 2001 and was suspended on that date. In order to extend the Program beyond July 11, 2001, CL&P is now seeking authority to continue the actions set forth in (v) above, and any other aspect of the proposed transactions for which approval may be necessary, through July 8, 2004, the proposed date of expiration of the extended Program.

The Programs consists of two agreements. As extended to July 8, 2004, the Program will continue in place with the same provisions set present. The principal features of the Program are as follows: under the first agreement, between CL&P and CRC ("Company Agreement"), CL&P sells or transfers as equity contributions from time to time