

*Effective date:* As of the date of issuance and shall be implemented 90 days from the date of issuance.

*Amendment No.:* 228.

*Facility Operating License No. NPF-38:* The amendment revised the Facility Operating License and Technical Specifications.

*Date of initial notice in Federal Register:* December 15, 2009 (74 FR 66384).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 13, 2010.

*No significant hazards consideration comments received:* No.

**Exelon Generation Company, LLC, Docket Nos. STN 50-456 and STN 50-457, Braidwood Station, Units 1 and 2 (Braidwood), Will County, Illinois; Docket Nos. STN 50-454 and STN 50-455, Byron Station, Unit Nos. 1 and 2 (Byron), Ogle County, Illinois**

*Date of application for amendment:* March 29, 2010.

*Brief description of amendment:* The amendments revise Technical Specification (TS) 5.5.7, "Reactor Coolant Pump Flywheel Inspection Program," to extend the reactor coolant pump (RCP) motor flywheel examination frequency from the currently-approved 10-year inspection interval to an interval not to exceed 20 years for certain Braidwood and Byron RCPs. These changes are consistent with TS Task Force (TSTF) traveler TSTF-421, "Revision to RCP Flywheel Inspection Program (WCAP-15666)," Revision 0, that has been approved generically for the Westinghouse Standard Technical Specifications, NUREG-1431. A notice announcing the availability of this proposed TS change using the Consolidated Line Item Improvement Process was published in the **Federal Register** on October 22, 2003 (68 FR 60422).

*Date of issuance:* September 16, 2010.

*Effective date:* As of the date of issuance and shall be implemented within 60 days.

*Amendment Nos.:* Braidwood Unit 1-163; Braidwood Unit 2-163; Byron Unit No. 1-169; and Byron Unit No. 2-169.

*Facility Operating License Nos. NPF-72, NPF-77, NPF-37, and NPF-66:* The amendments revise the TSs and Licenses.

*Date of initial notice in Federal Register:* May 18, 2010 (75 FR 27827).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 16, 2010.

*No significant hazards consideration comments received:* No.

**Exelon Generation Company, LLC, Docket No. 50-461, Clinton Power Station, Unit No. 1, DeWitt County, Illinois**

*Date of application for amendment:* April 2, 2010, as supplemented by letters dated June 19, 2009, and March 31, 2010.

*Brief description of amendment:* The amendment revises the Exelon Nuclear Radiological Emergency Plan Annex for Clinton Station, Table B-1, "Minimum Staffing Requirements for the On-Shift Clinton Station ERO," to increase the Non-Licensed Operator staffing from two to four, allow in-plant protective actions to be performed by personnel assigned to other functions, and replace a Mechanical Maintenance person with a Non-Licensed Operator.

*Date of issuance:* September 21, 2010.

*Effective date:* As of the date of issuance and shall be implemented within 30 days.

*Amendment No.:* 191.

*Facility Operating License No. NPF-62:* The amendment revised the Facility Operating License.

*Date of initial notice in Federal Register:* June 1, 2010 (75 FR 30445).

The June 19, 2009, and March 31, 2010, supplement, contained clarifying information and did not change the NRC staff's initial proposed finding of no significant hazards consideration.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated September 21, 2010.

*No significant hazards consideration comments received:* No.

**Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois**

*Date of application for amendments:* January 27, 2010, as supplemented by letters dated May 12, and May 13, 2010.

*Brief description of amendments:* The amendments would revise the Operating License and technical Specifications to implement an increase of approximately 1.65 percent in rated thermal power from the current licensed thermal power of 3489 megawatts thermal (MWt) to 3546 MWt.

*Date of issuance:* September 16, 2010.

*Effective date:* As of the date of issuance and shall be implemented within 90 days for Unit 1 and within 90 days of completion of refueling outage L2R13, which is currently scheduled for March 2011, for Unit 2.

*Amendment Nos.:* 198/185.

*Facility Operating License Nos. NPF-11 and NPF-18:* The amendments revised the Technical Specifications and License.

*Date of initial notice in Federal Register:* May 11, 2010 (75 FR 26289).

The May 12, and May 13, 2010, supplements, contained clarifying information and did not change the NRC staff's initial proposed finding of no significant hazards consideration.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated September 16, 2010.

*No significant hazards consideration comments received:* No.

Dated at Rockville, Maryland, this 23rd day of September 2010.

For the Nuclear Regulatory Commission.

**Joseph G. Giitter,**

*Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2010-24815 Filed 10-4-10; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2010-0312]

### Issuance of Regulatory Guides

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) is providing notice of the issuance and availability of Regulatory Guides 1.84, Rev. 35, "Design, Fabrication, and Materials Code Case Acceptability, ASME Section III," and RG 1.147, Rev. 16, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1."

**FOR FURTHER INFORMATION CONTACT:** Wallace E. Norris, Component Integrity Branch, Division of Engineering, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 251-7650 or e-mail [Wallace.Norris@nrc.gov](mailto:Wallace.Norris@nrc.gov).

### SUPPLEMENTARY INFORMATION:

#### I. Introduction

The NRC is issuing two final Regulatory Guides (RGs) in the agency's "Regulatory Guide" series: RG 1.84 and RG 1.147. This series was developed to describe and make available to the public specific program information. This information includes methods acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques the staff uses in evaluating specific problems or postulated accidents, and data the staff needs in its review of applications for permits and licenses.

The NRC published a proposed rule that would incorporate by reference RG 1.84, Revision 35, and RG 1.147, Revision 16, on June 2, 2009, 74 FR 26303. On the same date, the NRC published a parallel notice of availability for the draft regulatory guides and opportunity for public comment. See, 74 FR 26440. The NRC provided a 75-day public comment period for both the proposed rule and the draft RGs, which ended on August 17, 2009.

This Notice of Issuance and Availability addresses final RGs 1.84 and 1.147. The final rule that incorporates RG 1.84 and RG 1.147 by reference into Title 10, part 50, of the *Code of Federal Regulations* (10 CFR Part 50), "Domestic Licensing of Production and Utilization Facilities" contains the regulatory analysis and responses to the public comments relative to the final RGs and is available in the NRC's Agencywide Documents Access and Management System (ADAMS) under Accession No. ML100560148. For further information on the final rule, contact Manash K. Bagchi, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 415-2905, e-mail [Manash.Bagchi@nrc.gov](mailto:Manash.Bagchi@nrc.gov).

This action allows nuclear power plant licensees, and applicants for standard design certifications, standard design approvals, and manufacturing licenses under the regulations that govern license certifications, and approves the nuclear power plants to use the Code Cases listed in these RGs as alternatives to requirements in the ASME Boiler and Pressure Vessel (BPV) Code regarding the construction and in-service inspection (ISI) of nuclear power plant components. RG 1.84 lists all Section III Code Cases that NRC has approved for use, and RG 1.147 lists all Section XI Code Cases that have been approved for use. For these RG revisions, NRC reviewed the Code Cases listed in Supplements 2B11 to the 2004 Edition of the ASME BPV Code and Supplement 0 to the 2007 Edition (Supplement 0 also serves as Supplement 12 to the 2004 Edition).

## II. Further Information

Copies of the RGs are available in ADAMS under Accession Numbers ML101800532 (RG 1.84) and ML101800536 (RG 1.147). Electronic copies of RG 1.84 and RG 1.147 are available through the NRC's public Web site under "Regulatory Guides" at <http://www.nrc.gov/reading-rm/doc-collections/>.

In addition, RGs are available for inspection at NRC's Public Document Room (PDR) located at 11555 Rockville Pike, Rockville, Maryland. The PDR's mailing address is USNRC PDR, Washington, DC 20555-0001. The PDR also can be reached by telephone at (301) 415-4737 or (800) 397-4205, by fax at (301) 415-3548, and by e-mail to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

Regulatory guides are not copyrighted and NRC approval is not required to reproduce them.

Dated at Rockville, Maryland, this 23rd day of September, 2010.

For the U.S. Nuclear Regulatory Commission.

**Brian W. Sheron,**

*Director, Office of Nuclear Regulatory Research.*

[FR Doc. 2010-24812 Filed 10-4-10; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2010-0311]

### Issuance of Regulatory Guide

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice.

**SUMMARY:** The Nuclear Regulatory Commission (NRC) is providing notice of the issuance and availability of Regulatory Guide 1.193, Rev. 3, "ASME Code Cases Not Approved for Use."

**FOR FURTHER INFORMATION CONTACT:** Wallace E. Norris, Component Integrity Branch, Division of Engineering, Office of Nuclear Regulatory Research, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 251-7650 or e-mail [Wallace.Norris@nrc.gov](mailto:Wallace.Norris@nrc.gov).

### SUPPLEMENTARY INFORMATION:

#### I. Introduction

The NRC is issuing Regulatory Guide (RG) 1.193 in the agency's "Regulatory Guide" series. This series was developed to describe and make available to the public specific program information. This information includes methods acceptable to the NRC staff for implementing specific parts of the agency's regulations, techniques the staff uses in evaluating specific problems or postulated accidents, and data the staff needs in its review of applications for permits and licenses.

#### II. Discussion

RG 1.193 lists the Code Cases that NRC has determined not to be acceptable for use on a generic basis.

Two separate RGs list Code Cases that NRC has found to be acceptable alternatives to requirements in the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code. RG 1.84, Rev. 35, "Design, Fabrication, and Materials Code Case Acceptability, ASME Section III," lists all Section III Code Cases that NRC has approved for use. RG 1.147, Rev. 16, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1," lists all Section XI Code Cases that NRC has approved for use. RG 1.84 and RG 1.147 are being noticed under a separate notice of availability.

For Revision 3 of RG 1.193, NRC reviewed the Code Cases listed in Supplements 2-11 to the 2004 Edition of the ASME Boiler and Pressure Vessel (BPV) Code and Supplement 0 to the 2007 Edition (Supplement 0 also serves as Supplement 12 to the 2004 Edition). A brief description of the basis for the unacceptability determination is provided with each Code Case in RG 1.193. Licensees may submit a request to implement one or more of the Code Cases listed in RG 1.193 through 10 CFR 50.55a(a)(3), which permits the use of alternatives to the Code requirements referenced in 10 CFR 50.55a provided that the proposed alternatives result in an acceptable level of quality and safety. Licensees must submit a plant-specific request that addresses NRC's concerns about the Code Case at issue.

### III. Further Information

In June 2009, draft Regulatory Guide (DG) 1191, DG 1192, and DG 1193 were published with a public comment period of 60 days from their issuance. The comment period closed on August 16, 2009. Copies of the final RGs are available in ADAMS under Accession Numbers ML101800532 (RG 1.84), ML101800536 (RG 1.147), and ML101800540 (RG 1.193).

The responses to the public comments on the three RGs are contained in the final rule that incorporates RG 1.84 and RG 1.147 by reference into Title 10, part 50, of the *Code of Federal Regulations* (10 CFR part 50), "Domestic Licensing of Production and Utilization Facilities," and is available in NRC's Agencywide Documents Access and Management System (ADAMS) under Accession No. ML100560148. For further information on the final rule, contact Manash K. Bagchi, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, telephone (301) 415-2905, e-mail [Manash.Bagchi@nrc.gov](mailto:Manash.Bagchi@nrc.gov).

Electronic copies of RG 1.84, RG 1.147, and RG 1.193 are available through NRC's public Web site under