

**NATIONAL SCIENCE FOUNDATION****Agency Information Collection  
Activities: Comment Request****AGENCY:** National Science Foundation.**ACTION:** Submission for OMB review; comment request.

**SUMMARY:** The National Science Foundation (NSF) has submitted the following information collection requirements to OMB for review and clearance under the Paperwork Reduction Act of 1995, Public Law 104–13. This is the second notice for public comment; the first was published in the *Federal Register* at 79 FR 69883, and no comments were received. NSF is forwarding the proposed renewal submission to the Office of Management and Budget (OMB) for clearance simultaneously with the publication of this second notice. The full submission may be found at: <http://www.reginfo.gov/public/do/PRAMain>.

**Comments:** Comments regarding (a) whether the collection of information is necessary for the proper performance of the functions of the agency, including whether the information will have practical utility; (b) the accuracy of the agency's estimate of burden including the validity of the methodology and assumptions used; (c) ways to enhance the quality, utility and clarity of the information to be collected; (d) ways to minimize the burden of the collection of information on those who are to respond, including through the use of appropriate automated, electronic, mechanical, or other technological collection techniques or other forms of information technology should be addressed to: Office of Information and Regulatory Affairs of OMB, Attention: Desk Officer for National Science Foundation, 725—17th Street NW., Room 10235, Washington, DC 20503, and to Suzanne H. Plimpton, Reports Clearance Officer, National Science Foundation, 4201 Wilson Boulevard, Suite 1265, Arlington, Virginia 22230 or send email to [splimpto@nsf.gov](mailto:splimpto@nsf.gov). Comments regarding these information collections are best assured of having their full effect if received within 30 days of this notification. Copies of the submission(s) may be obtained by calling 703–292–7556.

It is not permissible for NSF to conduct or sponsor a collection of information unless the collection of information displays a currently valid OMB control number and the agency informs potential persons who are to respond to the collection of information that such persons are not required to respond to the collection of information

unless it displays a currently valid OMB control number.

**SUPPLEMENTARY INFORMATION:***Title:* Survey of Earned Doctorates.*OMB Control Number:* 3145–0019.

*Summary of Collection:* The authority to collect information for the Survey of Earned Doctorates (SED) is established under the National Science Foundation Act of 1950, as amended, Public Law 507 (42 U.S.C. 1862), Section 3(a) (6), which directs the NSF “. . . to provide a central clearinghouse for the collection, interpretation, and analysis of data on scientific and engineering resources and to provide a source of information for policy formation by other agencies of the federal government.” More recently, the National Center for Science and Engineering Statistics (NCSES) was established within NSF by Section 505 of the America COMPETES Reauthorization Act of 2010 and given a broader mandate to collect data related to STEM education, the science and engineering workforce, and U.S. competitiveness in science, engineering, technology, and R&D. The SED is part of an integrated survey system that fulfills the education and workforce components of this mission.

The SED has been conducted annually since 1958 and is jointly sponsored by six Federal agencies (NSF, National Institutes of Health, U.S. Department of Education, U.S. Department of Agriculture, National Endowment for the Humanities, and National Aeronautics and Space Administration) in order to avoid duplication of effort in collecting such data. It is an accurate, timely source of information on an important national resource—highly educated individuals. Data are obtained via Web survey or paper questionnaire from each person earning a research doctorate at the time they receive the degree. Graduate schools help distribute the SED to their graduating doctorate recipients. Data are collected on the doctorate recipient's field of specialty, educational background, sources of financial support in graduate school, debt level, postgraduation plans for employment, and demographic characteristics.

The survey will be collected in conformance with the National Science Foundation Act of 1950, as amended, and the Privacy Act of 1974. Responses from individuals are voluntary. NSF will ensure that all individually identifiable information collected will be kept strictly confidential and will be used for research or statistical purposes, analyzing data, and preparing scientific reports and articles.

*Need and Use of the Information:* The Federal government, universities, researchers, and others use the information extensively. NSF, as the lead agency, publishes statistics from the survey in several reports, but primarily in the annual publication series, “Doctorate Recipients from U.S. Universities.” These reports are available on the Web. NSF uses this information to prepare congressionally mandated reports such as *Science and Engineering Indicators* and *Women, Minorities and Persons with Disabilities in Science and Engineering*.

*Description of Respondents:*  
Individuals.

*Number of Respondents:* 52,000.*Frequency of Responses:* Annually.*Total Burden Hours:* 28,800.

Dated: March 19, 2015.

**Suzanne H. Plimpton,***Reports Clearance Officer, National Science Foundation.*

[FR Doc. 2015–06672 Filed 3–23–15; 8:45 am]

**BILLING CODE 7555–01–P****NUCLEAR REGULATORY  
COMMISSION****[Docket No. 50–313; NRC–2015–0069]****Entergy Operations, Inc., Arkansas  
Nuclear One, Unit 1****AGENCY:** Nuclear Regulatory Commission.**ACTION:** Exemption; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to a March 20, 2014, request from Entergy Operations, Inc. (Entergy or the licensee), from the requirements to use Charpy V-notch ( $C_V$ ) and drop weight-based methodology to determine initial nil-ductility reference temperature ( $RT_{NDT}$ ) for use in evaluating the integrity of Linde 80 weld materials in the reactor pressure vessel (RPV) beltline at Arkansas Nuclear One (ANO), Unit 1. This exemption would allow the licensee to use an alternate methodology to incorporate fracture toughness test data to determine  $RT_{NDT}$  values for use in the evaluation of the RPV beltline weld material integrity in support of the development of updated pressure-temperature limit curves.

**DATES:** March 24, 2015.

**ADDRESSES:** Please refer to Docket ID NRC–2015–0069 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- Federal Rulemaking Web site: Go to <http://www.regulations.gov> and search for Docket ID NRC–2015–0069. Address questions about NRC dockets to Carol Gallagher; telephone: 301–415–3463; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- NRC's Agencywide Documents Access and Management System (ADAMS): You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that a document is referenced.

- NRC's PDR: You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

**FOR FURTHER INFORMATION CONTACT:** Andrea George, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–1081, email: [Andrea.George@nrc.gov](mailto:Andrea.George@nrc.gov).

**SUPPLEMENTARY INFORMATION:**

### I. Background

Entergy is the holder of renewed Facility Operating License No. DPR–51, that authorizes operation of ANO, Unit 1. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the NRC now or hereafter in effect.

The ANO facility consists of two pressurized-water reactors, Units 1 and 2, located in Pope County, Arkansas.

### II. Request/Action

Part 50 of title 10 of the *Code of Federal Regulation* (10 CFR), appendix G, "Fracture Toughness Requirements," specifies fracture toughness requirements for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary of light water reactors to provide adequate margins of safety during any condition of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected to over its service lifetime. Section 50.61,

"Fracture toughness requirements for protection against pressurized thermal shock [PTS] events," provides fracture toughness requirements for protection against PTS events. A PTS event is an event or transient in pressurized water reactors (PWRs) causing severe overcooling (thermal shock) concurrent with or followed by significant pressure in the reactor vessel. Pursuant to 10 CFR 50.12, "Specific exemptions," by letter dated March 20, 2014 (ADAMS Accession No. ML14083A640), as supplemented by letter dated June 26, 2014 (ADAMS Accession No. ML14177A302), the licensee requested an exemption from certain requirements of 10 CFR part 50, appendix G, and 10 CFR 50.61, to revise certain ANO, Unit 1 RPV initial (unirradiated) properties using AREVA Topical Report (TR) BAW–2308, Revisions 1–A and 2–A, "Initial RTNDT [nil-ductility reference temperature] of Linde 80 Weld Materials."

Specifically, the licensee requested an exemption from 10 CFR part 50, appendix G.II.D(i), which requires that licensees evaluate the pre-service or unirradiated RT<sub>NDT</sub> according to the procedures in the American Society of Mechanical Engineers (ASME) Code, Paragraph NB–2331, "Material for Vessels." The ASME Code Paragraph NB–2331 requires that licensees use Charpy V-notch (C<sub>v</sub>) and drop weight-based methodology to derive the initial RT<sub>NDT</sub> values. In lieu of the existing methodology described above, the licensee requested to use the alternate methodology in TR BAW–2308, Revisions 1–A and 2–A, to incorporate the use of fracture toughness test data for evaluating the integrity of the ANO, Unit 1, Linde 80 weld materials in the RPV beltline. The methodology in TR BAW–2308, Revisions 1–A and 2–A, is based on the use of the 1997 and 2002 editions of the American Society for Testing and Materials (ASTM) Standard Test Method E1921 (ASTM E1921), "Standard Test Method for Determination of Reference Temperature T<sub>0</sub> for Ferritic Steels in the Transition Range," and ASME Code Case N–629, "Use of Fracture Toughness Test Data to Establish Reference Temperature for Pressure Retaining Materials, Section III, Division 1, Class 1." Since the licensee is proposing an alternate method to the C<sub>v</sub> and drop weight-based test data required by procedures in the ASME Code, Paragraph NB–2331, an exemption from portions of 10 CFR part 50, appendix G, is required.

The licensee also requested an exemption from 10 CFR 50.61(a)(5), which defines the method for evaluating

initial (unirradiated) RT<sub>NDT</sub> as one that uses the procedures in ASME Code, Paragraph NB–2331, which requires the use of C<sub>v</sub> and drop weight-based test data. 10 CFR 50.61(a)(5) alternatively defines the method for evaluating RT<sub>NDT</sub> as a method other than that of ASME Code, Paragraph NB–2331 approved by the Director, Office of Nuclear Reactor Regulation (NRR). The licensee proposes to use the alternate methodology described above, in AREVA TR BAW–2308," Revisions 1–A and 2–A, to determine the initial RT<sub>NDT</sub> values for the Linde 80 weld materials present in the ANO, Unit 1, RPV beltline region, which is not the procedure in ASME Code, Paragraph NB–2331 or an alternative method approved by the Director of NRR. Therefore, an exemption from 10 CFR 50.61(a)(5) is required.

### III. Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR part 50 when: (1) The exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. Under 10 CFR 50.12(a)(2)(ii), special circumstances include, among other things, when application of the specific regulation in the particular circumstance would not serve, or is not necessary to achieve, the underlying purpose of the rule.

#### A. Authorized by Law

As stated above, 10 CFR 50.12(a) allows the NRC to grant exemptions from portions of the requirements of 10 CFR part 50, appendix G and 10 CFR 50.61. Moreover, Section 50.60(b) of 10 CFR part 50 specifically allows the use of alternative methods for determining the initial material properties to 10 CFR part 50, appendix G, or portions thereof, when an exemption is granted by the Commission under 10 CFR 50.12. Because the regulations contemplate exemptions, granting the licensee's proposed exemption will not result in a violation of the Atomic Energy Act of 1954, as amended, or the NRC's regulations. Finally, this exemption would allow the licensee to make use of fracture toughness test data for evaluating the integrity of the ANO, Unit 1 RPV Linde 80 beltline weld materials, and would not result in changes to the operation of the plant. Therefore, the exemption is authorized by law.

### C. No Undue Risk to Public Health and Safety

The underlying purpose of appendix G to 10 CFR part 50 is to set forth fracture toughness requirements for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary of light-water nuclear power reactors to provide adequate margins of safety during any conditions of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime. The methodology underlying the requirements of appendix G to 10 CFR part 50 is based on the use of  $C_V$  and drop weight test data because of the reference to the ASME Code, Section III, Paragraph NB-2331. The licensee proposes to replace the use of existing  $C_V$  and drop weight-based methodology with an alternate methodology that uses fracture toughness test data to demonstrate compliance with appendix G to 10 CFR part 50. The alternate method, described in AREVA TR BAW-2308, Revisions 1-A and 2-A, utilizes fracture toughness data to determine the initial  $RT_{NDT}$  of the Linde 80 weld materials present in the ANO, Unit 1 RPV beltline.

The NRC staff has concluded that the requested exemption to Appendix G to 10 CFR part 50 is justified because the licensee will utilize the fracture toughness methodology specified in BAW-2308, Revisions 1-A and 2-A, within the conditions and limitations delineated in the NRC staff's safety evaluations (SEs) dated August 4, 2005, and March 24, 2008 (ADAMS Accession Nos. ML052070408 and ML080770349, respectively). The use of the methodology specified in the NRC staff's SEs will ensure that pressure-temperature limits developed for the ANO, Unit 1 RPV will continue to be based on an adequately conservative estimate of RPV material properties and ensure that the pressure-retaining components of the reactor coolant pressure boundary retain adequate margins of safety during any condition of normal operation, including anticipated operational occurrences. This exemption only modifies the methodology to be used by the licensee under 10 CFR part 50, appendix G.II.D(i) and does not exempt the licensee from meeting any other requirement of appendix G to 10 CFR part 50.

Based on the above information, no new accident precursors are created by allowing an exemption from the use of the existing  $C_V$  and drop weight-based methodology and the use of an

alternative fracture toughness-based methodology to demonstrate compliance with appendix G to 10 CFR part 50; thus, the probability of postulated accidents is not increased. Also, based on the above information, the consequences of postulated accidents are not increased. Therefore, there is no undue risk to public health and safety associated with the proposed exemption to appendix G to 10 CFR part 50.

The underlying purpose of 10 CFR 50.61 is to establish requirements for evaluating the fracture toughness of RPV materials to ensure that a licensee's RPV will be protected from failure during a PTS event. The licensee seeks an exemption from portions of 10 CFR 50.61 to use a methodology for the determination of adjusted/indexing PTS reference temperature ( $RT_{PTS}$ ) values. The licensee proposes to use the methodology of TR BAW-2308, Revisions 1-A and 2-A as an alternative to the  $C_V$  and drop weight-based methodology required by 10 CFR 50.61 for determining the initial, unirradiated properties when calculating  $RT_{PTS}$ . The NRC has concluded that the exemption is justified because the licensee will utilize the methodology specified in the NRC staff's SEs regarding TR BAW-2308, Revisions 1-A and 2-A.

In TR BAW-2308, Revision 1-A, the Babcock and Wilcox Owners Group proposed to perform fracture toughness testing based on the application of the Master Curve evaluation procedure, which permits data obtained from sample sets tested at different temperatures to be combined, as the basis for defining the initial material properties of Linde 80 welds based on  $T_0$  (initial temperature). The NRC staff evaluated this methodology for determining Linde 80 weld initial material properties and uncertainty in those properties, as well as the overall method for combining initial material property measurements based on  $T_0$  values (*i.e.* initial unirradiated nil-ductility reference temperature ( $IRT_{T_0}$ ) in the BAW-2308 terminology), with property shifts from models in Regulatory Guide (RG) 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials," which are based on  $C_V$  testing and defined margin term to account for uncertainties in the NRC staff's SE for TR BAW-2308, Revision 1-A. In the same NRC staff SE., Table 3, "NRC Staff-Accepted  $IRT_{T_0}$  and [Initial Margin]  $\sigma_i$  Values for Linde 80 Weld Wire Heats," contains the NRC staff's accepted  $IRT_{T_0}$  and initial margin (denoted as  $\sigma_i$ ) for specific Linde 80 weld wire heat numbers.

In accordance with the limitations and conditions outlined in the NRC staff's SE for TR BAW-2308, Revision 1-A, for utilizing the values in Table 3: The licensee has (1) utilized the appropriate NRC staff-accepted  $IRT_{T_0}$  and  $\sigma_i$  values for applicable Linde 80 weld wire heat numbers; (2) applied a minimum chemistry factor of 167 degrees Fahrenheit ( $^{\circ}F$ ) (values greater than 167  $^{\circ}F$  were used for certain Linde 80 weld wire heat numbers if RG 1.99, Revision 2 indicated higher chemistry factors); (3) applied a value of 28  $^{\circ}F$  for  $\sigma_{\Delta}$  (*i.e.*, shift margin) in the margin term; and (4) submitted values for  $\Delta RT_{NDT}$  and the margin term for each Linde 80 weld in the RPV though the end of the current operating license. Additionally, the NRC's SE for TR BAW-2308, Revision 2-A concludes that the revised  $IRT_{T_0}$  and  $\sigma_i$  values for Linde 80 weld materials are acceptable for referencing in plant-specific licensing applications as delineated in TR BAW-2308, Revision 2-A and to the extent specified under Section 4.0, "Limitations and Conditions," of the SE. Incidentally, although Section 4.0 of the NRC staff SE states "Future plant-specific applications for RPVs containing weld heat 72105, and weld heat 299L44, of Linde 80 must use the revised  $IRT_{T_0}$  and  $\sigma_i$  values in TR BAW-2308, Revision 2," the NRC notes that neither of these weld heats is used at ANO, Unit 1. Therefore, this condition does not apply to ANO, Unit 1.

During review of the licensee's exemption request, the NRC staff noted that additional information was required in order to complete its review regarding the chemistry factors used by the licensee for calculating  $\Delta RT_{NDT}$  values. The NRC staff requested this additional information via letter dated June 4, 2014 (ADAMS Accession No. ML14148A382). In the licensee's supplement dated June 26, 2014, the licensee provided the chemistry factors in Table 1, "10 CFR 50.61 Chemistry Factors for the ANO-1 RV [Reactor Vessel] Materials." The NRC staff confirmed that the chemistry factors used by the licensee in calculating the  $RT_{NDT}$  values were determined using the methodology of RG 1.99, Revision 2, and that 167  $^{\circ}F$  is the minimum chemistry factor for Linde 80 materials.

The use of the methodology in TR BAW-2308, Revisions 1-A and 2-A, will ensure the PTS evaluation developed for the ANO, Unit 1 RPV will continue to be based on an adequately conservative estimate of RPV material properties and ensure that the RPV will be protected from failure during a PTS event. Based on the evaluations above, the NRC staff has concluded that all

conditions and limitations outlined in the NRC staff's SEs for TR BAW-2308, Revisions 1-A and 2-A, have been met for ANO Unit 1.

Based on the above information, no new accident precursors are created by allowing an exemption to the alternate methodology to comply with the requirements of 10 CFR 50.61 in determining adjusted/indexing reference temperatures; thus, the probability of postulated accidents is not increased. Also, based on the above information, the consequences of postulated accidents are not increased. Therefore there is no undue risk to public health and safety.

#### *D. Consistent With the Common Defense and Security*

The licensee requested an exemption in order to utilize an alternative methodology from that specified in portions of 10 CFR part 50, appendix G, and 10 CFR 50.61, to allow the use of fracture toughness test data for evaluating the integrity of the ANO, Unit 1 RPV beltline Linde 80 weld materials. This exemption request is not related to, and does not impact, any security issues at ANO, Unit 1. Therefore, the NRC has determined that this exemption does not impact, and is consistent with, the common defense and security.

#### *E. Special Circumstances*

Special circumstances, in accordance with 10 CFR 50.12(a)(2)(ii), are present whenever application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule. The underlying purpose of 10 CFR 50.61(a)(5) and 10 CFR part 50, appendix G.II.D(i) is to set forth fracture toughness requirements (e.g., initial  $RT_{NDT}$  values) for ferritic materials of pressure-retaining components of the reactor coolant pressure boundary of light water nuclear power reactors, in order to provide adequate margins of safety during any conditions of normal operation, including anticipated operational occurrences and system hydrostatic tests, to which the pressure boundary may be subjected over its service lifetime. The underlying purpose of 10 CFR 50.61 is to establish requirements for evaluating the fracture toughness of RPV materials to ensure that a licensee's RPV will be protected from failure during a PTS event.

Entergy's exemption request proposes an alternate methodology to evaluate the  $RT_{NDT}$  of Linde 80 weld materials in the RPV beltline region at ANO, Unit 1, based on fracture toughness test data

found in AREVA TR BAW-2308, Revision 1-A and 2-A (in accordance with ASTM Standard E1921 and ASME Code Case N-629). This proposed alternate methodology achieves the underlying purpose of 10 CFR part 50 appendix G.II.D(i) because it provides an adequate conservative estimate of RPV materials properties and ensures that the pressure-retaining components of the RPV retain adequate margins for safety during any condition of normal operation. The alternate methodology also achieves the underlying purpose of 10 CFR 50.61(a)(5) because it will ensure that the PTS evaluation developed for the ANO, Unit 1 RPV will continue to be based on an adequately conservative estimate of RPV material properties and ensure that the RPV will be protected from failure during a PTS event. Accordingly, the NRC has concluded that using the procedures in the ASME Code, Paragraph NB-2331 is not necessary to achieve the underlying purpose of 10 CFR 50.61(a)(5) and 10 CFR part 50 appendix G.II.D(i). Therefore, the special circumstances required by 10 CFR 50.12(a)(2)(ii) for the granting of an exemption exist.

#### *F. Environmental Considerations*

The NRC staff determined that the exemption discussed herein meets the eligibility criteria for the categorical exclusion set forth in 10 CFR 51.22(c)(9) because it is related to a requirement concerning the installation or use of a facility component located within the restricted area, as defined in 10 CFR part 20, and issuance of this exemption involves: (i) No significant hazards consideration, (ii) no significant change in the types or a significant increase in the amounts of any effluents that may be released offsite, and (iii) no significant increase in individual or cumulative occupational radiation exposure. Therefore, in accordance with 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC's consideration of this exemption request. The basis for the NRC staff's determination is discussed as follows with an evaluation against each of the requirements in 10 CFR 51.22(c)(9)(i)-(iii).

Requirements in 10 CFR 51.22(c)(9)(i)

The NRC evaluated whether the exemption involves no significant hazards consideration using the standards described in 10 CFR 50.92(c), as presented below:

1. Does the proposed exemption involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The exemption would allow the use of alternate methodologies from those specified in appendix G to 10 CFR part 50, and 10 CFR 50.61, to allow the use of fracture toughness test data for evaluating the integrity of RPV beltline welds. Use of the alternate methodology for determining the initial, unirradiated material reference temperatures of the Linde 80 weld materials present in the RPV beltline region will not result in changes in operation of configuration of the facility. The change in reactor vessel material initial properties will continue to satisfy the intent of 10 CFR 50, Appendix G, and 10 CFR 50.61. The change does not adversely affect accident initiators or pre-cursors, nor alter the design assumptions, conditions, or the manner in which the plant is operated and maintained. The change does not alter or prevent the ability of structures, systems or components from performing their intended function to mitigate the consequences of an initiating event with the assumed acceptance limits. There will be no adverse change to normal plant operating parameters, engineered safety feature actuation setpoints, accident mitigation capabilities, or accident analysis assumptions or inputs. The change does not affect the source term, containment isolation, or radiological release assumptions used in evaluating the radiological consequences of an accident previously evaluated. Further, the change does not increase the types or amounts of radioactive effluent that may be released offsite, nor significantly increase individual or cumulative occupational/public radiation exposures.

Therefore, the proposed exemption does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed exemption create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The exemption would allow the use of alternate methodologies from those specified in appendix G to 10 CFR part 50, and 10 CFR 50.61, to allow the use of fracture toughness test data for evaluating the integrity of RPV beltline welds. Use of the alternate methodology for determining the initial, unirradiated material reference temperatures of the Linde 80 weld materials present in the

RPV beltline region will not result in changes in operation or configuration of the facility. The change does not impose any new or different requirements or eliminate any existing requirements. The change is consistent with the current safety analysis assumptions and current plant operating practice. No new accident scenarios, transient precursors, failure mechanisms, or limiting single failures are introduced as a result of the proposed change. Equipment important to safety will continue to operate as designed. The change does not result in any event previously deemed incredible being more credible. The change does not result in any adverse conditions or result in any increase in the challenges to safety systems.

Therefore, this change does not create the possibility of a new or different kind of accident from an accident previously evaluated.

3. Does the proposed exemption involve a significant reduction in a margin of safety?

Response: No.

The proposed exemption does not alter safety limits, limiting safety system settings, or limiting conditions for operation. The setpoints at which protective actions are initiated are not altered by the change. There are no new or significant changes to initial conditions contributing to accident severity or consequences. The exemption will not otherwise affect plant protective boundaries, will not cause a release of fission products to the public, nor will it degrade the performance of any other structures, systems or components important to safety.

Therefore, the proposed exemption does not involve a significant reduction in a margin of safety.

Based on the above evaluation of the standards set forth in 10 CFR 50.92(c), the NRC concludes that the proposed exemption involves no significant hazards consideration. Accordingly, the requirements of 10 CFR 51.22(c)(9)(i) are met.

Requirements in 10 CFR 51.22(c)(9)(ii–iii)

The proposed exemption does not make any changes to the facility, equipment at the facility, or to fuel or core design. The proposed alternate methodology serves the same purpose as the requirements set forth in 10 CFR 50.61 and 10 CFR part 50, appendix G. Therefore, the NRC concludes that the exemption involves no significant change in the types or a significant increase in the amounts of any effluents that may be released offsite, and that

there is no significant increase in individual or cumulative public or occupational radiation exposure.

Therefore, the requirements of 10 CFR 51.22(c)(9)(ii–iii) are met.

#### Conclusion

Based on the above, the NRC concludes that the proposed exemption meets the eligibility criteria for the categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, in accordance with 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC's issuance of this exemption.

#### IV. Conclusions

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a), the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances are present. Therefore, the Commission hereby grants the licensee an exemption from 10 CFR part 50, appendix G.II.D(i) and 10 CFR 50.61(a)(5) requirements, in order to use the alternate methodology specified in AREVA TR BAW–2308, Revisions 1–A and 2–A, in lieu of the existing requirement to use  $C_V$  and drop weight-based methodologies to evaluate the initial (unirradiated)  $RT_{NDT}$  of the Linde 80 weld materials in the RPV beltline region at ANO, Unit 1.

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 16th day of March 2015.

For the Nuclear Regulatory Commission.

**Michele G. Evans,**

*Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.*

[FR Doc. 2015–06700 Filed 3–23–15; 8:45 am]

**BILLING CODE 7590–01–P**

#### NUCLEAR REGULATORY COMMISSION

[Docket No. 50–305; NRC–2015–0068]

#### Dominion Energy Kewaunee, Inc.; Kewaunee Power Station

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Exemption; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption from certain power reactor liability insurance requirements in response to a request from Dominion Energy Kewaunee, Inc. (DEK or the

licensee) dated March 20, 2014. This exemption would permit the licensee to reduce its primary offsite liability insurance and withdraw from participation in the secondary retrospective rating pool for deferred premium charges.

**DATES:** March 24, 2015.

**ADDRESSES:** Please refer to Docket ID NRC–2015–0068 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- Federal Rulemaking Web site: Go to <http://www.regulations.gov> and search for Docket ID NRC–2015–0068. Address questions about NRC dockets to Carol Gallagher; telephone: 301–415–3463; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- NRC's Agencywide Documents Access and Management System (ADAMS): You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The ADAMS accession number for each document referenced (if it available in ADAMS) is provided the first time that a document is referenced.

- NRC's PDR: You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

**FOR FURTHER INFORMATION CONTACT:** William Huffman, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–2046; email: [William.Huffman@nrc.gov](mailto:William.Huffman@nrc.gov).

#### SUPPLEMENTARY INFORMATION:

##### I. Background

The Kewaunee Power Station (KPS) facility is a decommissioning power reactor located on approximately 900 acres in Carlton (Kewaunee County), Wisconsin, 27 miles southeast of Green Bay, Wisconsin. The licensee, DEK, is the holder of the KPS Renewed Facility Operating License No. DPR–43. The license provides, among other things, that the facility is subject to all rules,